

1. Do the proposed changes involve a significant increase in the probability or consequences of an accident previously evaluated?

*Response:* No.

The administrative changes do not affect any existing limits, and accident initial conditions, probability, and assumptions remain as previously analyzed. The proposed change to the name of the onsite review committee or the version of the Regulatory Guide will have no significant effect on accident initiation frequency. The proposed changes do not invalidate the assumptions used in evaluating the radiological consequences of any accident. Therefore, the proposed changes do not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. Do the proposed changes create the possibility of a new or different kind of accident from any accident previously evaluated?

*Response:* No.

The proposed changes are administrative and do not introduce any new or different accident initiators. Therefore, the proposed changes do not create the possibility of a new or different kind of accident from any previously evaluated.

3. Do the proposed changes involve a significant reduction in a margin of safety?

*Response:* No.

The proposed changes are administrative and will not have a significant effect on any margin of safety. Therefore, the proposed changes do not involve a significant reduction in a margin of safety.

Based on the above, FENOC concludes that the proposed amendment presents no significant hazards consideration under the standards set forth in 10 CFR 50.92(c), and, accordingly, a finding of "no significant hazards consideration" is justified.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

*Attorney for licensee:* Mary E. O'Reilly, Attorney, FirstEnergy Corporation, 76 South Main Street, Akron, OH 44308.

*NRC Section Chief:* Anthony J. Mendiola.

*Indiana Michigan Power Company, Docket Nos. 50-315 and 50-316, Donald C. Cook Nuclear Plant, Units 1 and 2, Berrien County, Michigan*

*Date of amendment requests:* January 14, 2003.

*Description of amendment requests:* The proposed amendments would revise the Technical Specifications (TSs) for the control room emergency ventilation system (CREVS) such that movement of irradiated fuel assemblies will be allowed to commence with one

CREVS pressurization train inoperable, provided the appropriate TS Action requirements are implemented.

*Basis for proposed no significant hazards consideration determination:*

As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. Does the proposed change involve a significant increase in the probability of occurrence or consequences of an accident previously evaluated?

*Response:* No

*Probability of Occurrence of an Accident Previously Evaluated*

[Cook Nuclear Plant] CNP TS 3.0.4 requires that TS limiting conditions for operation be met without reliance on the Action statements prior to entering an Applicability condition. The proposed change to the CNP CREVS TS to allow an exception to TS 3.0.4 during movement of irradiated fuel assemblies does not affect any accident initiators or precursors. The CREVS function is purely mitigative. There is no design basis accident that is initiated by a failure of the CREVS function. An exception to TS 3.0.4 will not create any adverse interactions with other systems that could result in initiation of a design basis accident. Therefore, the probability of occurrence of an accident previously evaluated is not significantly increased.

*Consequences of an Accident Previously Evaluated*

The accident consequence that is relevant to the proposed change is the dose to control room personnel from a fuel handling accident. The CNP licensing basis analysis of a fuel handling accident has determined that the dose would be within the applicable limits of GDC 19. The current TS specify actions to be taken if one CREVS pressurization train is inoperable during movement of irradiated fuel assemblies. These actions provide assurance that the CREVS will perform its mitigating function as assumed in the accident analysis. Since the proposed change will continue to require these actions, the fuel handling accident analysis will remain valid. Therefore, the consequences of an accident previously analyzed are not significantly increased.

In summary, the probability of occurrence and the consequences of an accident previously evaluated are not significantly increased.

2. Does the proposed change create the possibility of a new or different kind of accident from any accident previously evaluated?

*Response:* No

The proposed change does not create any new or different accident initiators or precursors. The option to commence movement of irradiated fuel assemblies while relying on the provisions of the Action statement does not affect the manner in which any accident begins. The proposed change does not create any new accident scenarios and does not change the interaction

between the CREVS and any other system. Thus, the proposed change does not create the possibility of a new or different kind of accident from any previously evaluated.

3. Does the proposed change involve a significant reduction in a margin of safety?

*Response:* No

The margin of safety associated with the proposed change is that associated with the applicable control room dose limit specified by GDC 19. The proposed change will continue to require actions that assure the dose to control room personnel determined by the fuel handling accident analysis remains valid. Therefore, the proposed change does not involve a significant reduction in margin of safety.

In summary, based upon the above evaluation, [Indiana Michigan Power] I&M has concluded that the proposed change involves no significant hazards consideration under the standards set forth in 10 CFR 50.92(c), and, accordingly, a finding of "no significant hazards consideration" is justified.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment requests involve no significant hazards consideration.

*Attorney for licensee:* David W. Jenkins, Esq., 500 Circle Drive, Buchanan, MI 49107.

*NRC Section Chief:* L. Raghavan.

*South Carolina Electric & Gas Company, South Carolina Public Service Authority, Docket No. 50-395, Virgil C. Summer Nuclear Station, Unit No. 1, Fairfield County, South Carolina*

*Date of amendment request:* January 14, 2003.

*Description of amendment request:* The proposed one-time change revises the steam generator inservice inspection frequency requirements in Technical Specification 4.4.5.3.a for V.C. Summer Nuclear Station (VCSNS) immediately after refueling outage RF-12. The change would allow a 58-month maximum inspection interval after two inspections resulting in C-1 classification, rather than a 40-month maximum inspection interval. This change is proposed to eliminate premature/unnecessary steam generator inspections, due to a shortened operating cycle, which will result in significant dose and schedule impacts.

*Basis for proposed no significant hazards consideration determination:*

As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. Does the proposed change involve a significant increase in the probability or