

pressure, temperature, and temperature rate of change conditions that might cause undetected flaws to propagate and cause non-ductile failure of the reactor coolant pressure boundary. Thus, they ensure that an accident precursor is not likely. Hence, they are included in the TS as satisfying Criterion 2 of 10 CFR 50.36(c)(2)(ii). The relocation of the numerical value of these limits to a licensee-controlled document does not remove the existing TS requirement that the limits be met. The new TS administrative controls for the PTLR will ensure that only NRC-approved methods are used to calculate the actual limits to be applied. Thus, this relocation will not increase the probability of any accident previously evaluated.

The proposed changes do not alter the design assumptions, conditions, or configuration of the facility or the manner in which the facility is operated or maintained. The proposed changes will not affect any other System, Structure or Component (SSC) designed for the mitigation of previously analyzed events. The proposed changes do not affect the source term, containment isolation, or radiological release assumptions used in evaluating the radiological consequences of any accident previously evaluated. Thus, the proposed relocation of the existing numerical values and the updated figure for the RCS P/T limits based upon an NRC-approved methodology, to a licensee-controlled document (*i.e.*, the PTLR), with all the requisite TS restrictions placed upon it by NRC Generic Letter 96-03, as modified by TSTF-419, Rev. 0, will not increase the consequences of any previously evaluated accident.

(2) The proposed amendment will not create the possibility of a new or different kind of accident from any accident previously evaluated.

The proposed changes do not involve a physical alteration of the plant (*i.e.*, no new or different type of equipment will be installed) or a change in the methods governing normal plant operation. In addition, the changes do not impose any new or different requirements or eliminate any existing requirements. The changes do not alter assumptions made in the safety analysis. The proposed changes are consistent with the safety analysis assumptions and current plant operating practice. We are merely requesting to move the existing numerical values and the updated figure for the RCS P/T limits based upon an NRC-approved methodology, from the TS to a licensee-controlled document (*i.e.*, the PTLR), with all the requisite TS restrictions placed upon it by NRC Generic Letter 96-03, as modified by TSTF-419, Rev. 0.

Therefore, the proposed changes do not create the possibility of a new or different kind of accident from any previously evaluated.

(3) The proposed amendment will not involve a significant reduction in a margin of safety.

The proposed changes do not alter the manner in which Safety Limits, Limiting Safety System Settings or Limiting Conditions for Operation are determined. The setpoints at which protective actions are

initiated are not altered by the proposed changes. Sufficient equipment remains available to actuate upon demand for the purpose of mitigating an analyzed event. We are merely requesting to move the existing numerical values and the updated figure for the RCS P/T limits based upon an NRC-approved methodology, from the TS to a licensee-controlled document (*i.e.*, the PTLR), with all the requisite TS restrictions placed upon it by NRC Generic Letter 96-03, as modified by TSTF-419, Rev. 0. Thus, the proposed changes will not significantly reduce any margin of safety that currently exists.

Based upon the above, NMC [Nuclear Management Company] has determined that the proposed amendment will not involve a significant hazards consideration.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

*Attorney for licensee:* Mr. Alvin Gutterman, Morgan Lewis, 1111 Pennsylvania Avenue NW Washington, DC 20004.

*NRC Section Chief:* L. Raghavan.

*Omaha Public Power District, Docket No. 50-285, Fort Calhoun Station, Unit No. 1, Washington County, Nebraska*

*Date of amendment request:* January 27, 2003.

*Description of amendment request:* The proposed amendment would make administrative and editorial changes to the Fort Calhoun Station (FCS) Technical Specifications (TS) 1.3 Basis (1); 2.7(1)a; 2.7(1)b; 2.7(1)d; 2.7(1)i; 2.7 Basis; 3.0.2; Table 3-5, Item 11; and 3.5(3)ii. The proposed changes consist primarily of editorial and typographical changes or corrections.

*Basis for proposed no significant hazards consideration determination:* As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

The correction of typographical errors and clarification of specifications is not an initiator of any previously evaluated accident. The frequency or periodicity of performance of those surveillances affected by this change are not an initiator of any previously evaluated accident. The proposed changes will not prevent safety systems from performing their accident mitigation function as assumed in the safety analysis.

Therefore, this change does not involve a significant increase in the probability or

consequences of any accident previously evaluated.

2. The proposed change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

The proposed change only affects the technical specifications and does not involve a physical change to the plant. Modifications will not be made to existing components nor will any new or different types of equipment be installed. The proposed change corrects typographical errors, provides clarification as to applicable equipment and modifies the frequency of surveillances performed once per shift from 8 hours to 12 hours. This change will not alter assumptions made in safety analysis and licensing bases.

Therefore, this change does not create the possibility of a new or different kind of accident from any previously evaluated.

3. The proposed change does not involve a significant reduction in a margin of safety.

The proposed change corrects typographical errors, provides clarification as to applicable equipment, and modifies the frequency of surveillances performed once per shift from 8 hours to 12 hours. The decrease in frequency or periodicity of performance of these surveillances will also permit more efficient and more safely managed plant operations and can help reduce the risk associated with changing plant equipment or operating modes in order to obtain some of these readings.

Therefore, this technical specification change does not involve a significant reduction in the margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

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*NRC Section Chief:* Stephen Dembek.

*Omaha Public Power District, Docket No. 50-285, Fort Calhoun Station, Unit No. 1, Washington County, Nebraska*

*Date of amendment request:* January 27, 2003.

*Description of amendment request:* The proposed amendment would delete the allowance to perform the surveillance test of Table 3-2, Item 20 (Recirculation Actuation Logic Channel Functional Test) under administrative controls, while components in excess of those allowed by Conditions a, b, d, and e of Technical Specification 2.3(2) are inoperable provided they are returned to operable status within one hour. This allowance was granted in Amendment No. 206 issued April 19, 2002, on an exigent basis and applies only for the remainder of the current cycle. Omaha