

Public Power District committed to submit a permanent resolution to this allowance and this license amendment request constitutes this permanent resolution.

*Basis for proposed no significant hazards consideration determination:* As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

Deleting the requirement to perform the quarterly surveillance test of Table 3-2, Item 20 (Recirculation Actuation Logic Channel Functional Test) under administrative controls is acceptable since the performance of the recirculation actuation logic channel functional test is not identified as the initiator of any analyzed event. The proposed change will still require that the surveillance test be performed and the required ECCS [emergency core cooling system] systems to be available. This change will not alter assumptions relative to the mitigation of an accident or transient event. The performance of this activity has no effect on any accident scenario. Therefore, the proposed change does not involve a significant increase in the consequences of an accident previously evaluated.

2. The proposed change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

This change only removes a short term allowance to utilize administrative controls in the performance of the recirculation actuation logic channel functional test. These proposed changes do not involve a physical alteration of the plant (no new or different type of equipment will be installed) or change the methods governing plant operation. The proposed change does not involve any physical changes to plant systems, structures or components (SSCs) or the manner in which these SSCs are operated, maintained, modified or inspected. Therefore, these changes do not create the possibility of a new or different kind of accident from any accident previously evaluated.

3. The proposed change does not involve a significant reduction in a margin of safety.

The minimum numbers of ECCS components required by the FCS [Fort Calhoun Station] accident analyses will remain available. The proposed change to delete the short term allowance to utilize administrative controls in the performance of the recirculation actuation logic channel functional test will not significantly impact the availability or reliability of the plant's systems or their ability to respond to plant transients and accidents. The performance of this activity has no effect on any accident scenario. Therefore, the proposed changes do not involve a significant reduction in a margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

*Attorney for licensee:* James R. Curtiss, Esq., Winston & Strawn, 1400 L Street, NW., Washington, DC 20005-3502.

*NRC Section Chief:* Stephen Dembek.

*Omaha Public Power District, Docket No. 50-285, Fort Calhoun Station, Unit No. 1, Washington County, Nebraska*

*Date of amendment request:* January 27, 2003.

*Description of amendment request:* The proposed amendment would authorize the revision of the Fort Calhoun Station, Unit No. 1 Updated Safety Analysis Report (USAR). Section 14.16 and Figures 14.16-1 through 14.16-4 of the USAR will be revised to reflect the use of the GOTHIC, version 7.0, computer code and the results associated with the updated containment pressure analyses for a loss-of-coolant accident and main steam line break. In addition, GOTHIC will be used for the analysis of future plant upgrades associated with containment response and will be maintained consistent with other NRC-approved Omaha Public Power District methodologies.

*Basis for proposed no significant hazards consideration determination:* As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed changes will not increase the probability or consequence of any accident based on the following:

The proposed changes to Section 14.16 of the Updated Safety Analysis Report (USAR) and replacements for Figures 14.16-1 through 14.16-4 is required due to using GOTHIC, version 7.0 and the updated containment pressure analyses. Demonstrating that containment pressure is maintained less than the containment design pressure is required by Fort Calhoun Station (FCS) design basis. Additionally, the analyses credit all modes of heat transfer defined by Reference 10.5. Therefore, the updated containment pressure analyses using GOTHIC, version 7.0 is in compliance with FCS design basis. Changes to the containment pressure analyses for either a loss-of-coolant accident or main steam line break will be controlled by 10 CFR 50.59.

Therefore, the probability or consequence of any accident is not increased.

2. The proposed change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

The proposed revision does not change any equipment required to mitigate the consequences of an accident. The continued use of the same USAR administrative controls prevents the possibility of a new or different kind of accident. Since the proposed changes do not involve the addition or modification of equipment nor alter the design of plant systems, the proposed changes do not create the possibility of a new or different kind of accident from any accident previously evaluated. The changes proposed do not change how design basis accident events are postulated nor do the changes themselves initiate a new kind of accident or failure mode with a unique set of conditions (proposed administrative controls). Therefore, the proposed change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

3. The proposed change does not involve a significant reduction in a margin of safety.

The use of GOTHIC, version 7.0 is in compliance with FCS design basis. Additionally, GOTHIC has been benchmarked to the current analysis of record for a loss-of-coolant accident and main steam line break using the NRC approved computer code CONTRANS. These benchmark models demonstrate that GOTHIC provides similar results to CONTRANS. Future updates of the containment pressure analyses will be conducted under the 10 CFR 50.59 process. The analyses will credit all available modes of heat transfer defined by Reference 10.5. Additionally, the main steam line break containment evaluation model considers the leakage past the broken steam generator main feed isolation valve of 2.45% of full power flow or approximately 195 gpm. Therefore, the proposed changes do not involve a significant reduction to the margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

*Attorney for licensee:* James R. Curtiss, Esq., Winston & Strawn, 1400 L Street, NW., Washington, DC 20005-3502.

*NRC Section Chief:* Stephen Dembek.

*Omaha Public Power District, Docket No. 50-285, Fort Calhoun Station, Unit No. 1, Washington County, Nebraska*

*Date of amendment request:* January 27, 2003.

*Description of amendment request:* The proposed amendment revises Technical Specifications (TS) 2.1.6, 3.2