

*Duke Energy Corporation, Docket Nos. 50-269, 50-270, and 50-287, Oconee Nuclear Station, Units 1, 2, and 3, Oconee County, South Carolina*

*Date of amendment request:* February 17, 2003.

*Description of amendment request:* The proposed amendments would revise the Technical Specification Surveillance Requirement 3.10.1.9 to require that the Standby Shutdown Facility (SSF) diesel generator (DG) be loaded to at least 3280 kilowatts during the surveillance. The current requirement is that the SSF DG be loaded to at least 3000 kilowatts during the surveillance. The change supports resolution of an Oconee design basis issue associated with SSF pressurizer heater capacity.

*Basis for proposed no significant hazards consideration determination:* As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

Pursuant to 10 CFR 50.91, Duke Power Company (Duke) has made the determination that this amendment request involves a No Significant Hazards Consideration by applying the standards established by the NRC in 10 CFR 50.92. This ensures that operation of the facility in accordance with the proposed amendment would not:

1. Involve a significant increase in the probability or consequences of an accident previously evaluated:

This change revises the loading of the Standby Shutdown Facility (SSF) Diesel Generators (DG) to  $\geq 3280$  kW. The design rating of the DG is currently 3500 kW. Since the proposed loading is within the design rating already evaluated, this proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. Create the possibility of a new or different kind of accident from any kind of accident previously evaluated:

As stated above, the proposed revision revises the DG loading to an analytical value that is within the equipment's design limit. Applicable load and support system calculations have been revised and results have shown that the increase does not adversely affect the ability of the SSF diesel generator or SSF to perform its intended safety function. Additionally, this change is bounded by all of the existing accidents and does not create the possibility of a new or different kind of accident from any kind of accident previously evaluated.

3. Involve a significant reduction in a margin of safety.

The proposed change does not adversely affect any plant safety limits, set points, or design parameters. The change also does not adversely affect the fuel, fuel cladding, Reactor Coolant System, or containment integrity. Therefore, the proposed change does not involve a reduction in a margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

*Attorney for licensee:* Anne W. Cottoing, Winston and Strawn, 1200 17th Street, NW., Washington, DC 20005.

*NRC Section Chief:* John A. Nakoski.

*Exelon Generation Company, LLC, Docket Nos. 50-373 and 50-374, LaSalle County Station, Units 1 and 2, LaSalle County, Illinois*

*Date of amendment request:* January 31, 2003.

*Description of amendment request:* The proposed amendments would revise Appendix A, Technical Specifications (TS) 3.4.11, "RCS Pressure and Temperature (P/T) Limits," to incorporate revised P/T curves. The revised P/T curves are based on calculations performed in accordance with General Electric (GE) Topical Report NEDC-32983P, "General Electric Methodology for Reactor Pressure Vessel Fast Neutron Flux Evaluation." The NEDC-32983P methodology is consistent with the guidance contained in Regulatory Guide (RG) 1.190, "Calculational and Dosimetry Methods for Determining Pressure Vessel Neutron Fluence," dated March 2001.

*Basis for proposed no significant hazards consideration determination:* As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed changes request for LaSalle County Station, Units 1 and 2, that the pressure and temperature (P/T) limit curves in TS 3.4.11, "RCS Pressure and Temperature (P/T) Limits," and Surveillance Requirement (SR) 3.4.11.1 and SR 3.4.11.2 be revised. The revised curves were developed using the methodology of GE Topical Report NEDC-32983P, "General Electric Methodology for Reactor Pressure Vessel Fast Neutron Flux Evaluation." NEDC-32983P methodology has been previously approved by the NRC for use by licensees. The P/T limits are prescribed during normal operation to avoid encountering pressure, temperature, and temperature rate of change conditions that might cause undetected flaws to propagate and cause nonductile failure of the reactor coolant pressure boundary, a condition that is unanalyzed. Thus, the proposed changes

do not have any effect on the probability of an accident previously evaluated.

The P/T curves are used as operational limits during heatup or cooldown maneuvering, when pressure and temperature indications are monitored and compared to the applicable curve to determine that operation is within the allowable region. The P/T curves provide assurance that station operation is consistent with previously evaluated accidents. Thus, the radiological consequences of any accident previously evaluated are not increased.

Therefore, the proposed changes do not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. The proposed changes do not create the possibility of a new or different kind of accident from any accident previously evaluated.

The proposed changes do not change the response of plant equipment to transient conditions. The proposed changes do not introduce any new equipment, modes of system operation or failure mechanisms.

Therefore, the proposed changes do not create the possibility of a new or different kind of accident from any previously evaluated.

3. The proposed changes do not involve a significant reduction in a margin of safety.

The proposed changes adopt P/T curves that have been developed using the methodology of GE Topical Report NEDC-32983P. The NEDC-32983P methodology is consistent with the guidance contained in RG 1.190, "Calculational and Dosimetry Methods for Determining Pressure Vessel Neutron Fluence," dated March 2001. In a letter dated September 14, 2001, the NRC approved NEDC-32983P for use by licensees.

Therefore, the proposed changes do not involve a significant reduction in a margin of safety.

Based upon the above, EGC concludes that the proposed amendment presents no significant hazards consideration under the standards set forth in 10 CFR 50.92(c), and, accordingly, a finding of "no significant hazards consideration" is justified.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the requested amendments involve no significant hazards consideration.

*Attorney for licensee:* Mr. Edward J. Cullen, Deputy General Counsel, Exelon BSC—Legal, 2301 Market Street, Philadelphia, PA 19101.

*NRC Section Chief:* Anthony J. Mendiola.

*Exelon Generation Company, LLC, Docket Nos. 50-254 and 50-265, Quad Cities Nuclear Power Station, Units 1 and 2, Rock Island County, Illinois*

*Date of amendment request:* February 27, 2003.