

kind of accident from any accident previously evaluated. The changes in methods governing normal plant operation are consistent with the current safety analysis assumptions. Therefore, these proposed changes do not create the possibility of a new or different kind of accident from any accident previously evaluated.

3. Do the proposed changes involve a significant reduction in a margin of safety[?]  
*Response:* No.

Margins of safety are established in the design of components, the configuration of components to meet certain performance parameters, and in the establishment of setpoints to initiate alarms and actions. The proposed changes do not affect the probability of failure or availability of the affected instrumentation, and the proposed changes do not revise any allowable values for RPS functions. Therefore, it is concluded that the proposed changes do not result in a reduction in the margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the requested amendments involve no significant hazards consideration.

*Attorney for licensee:* Mr. Edward J. Cullen, Deputy General Counsel, Exelon BSC—Legal, 2301 Market Street, Philadelphia, PA 19101.

*NRC Section Chief:* Anthony J. Mendiola.

*FirstEnergy Nuclear Operating Company, Docket No. 50-440, Perry Nuclear Power Plant, Unit 1, Lake County, Ohio*

*Date of amendment request:* January 14, 2003.

*Description of amendment request:* This license amendment request proposes a change to Technical Specifications (TSs) 5.1.1, 5.4.1, and 5.5.1 that would replace the requirement for the plant manager to approve administrative procedures and the Offsite Dose Calculation Manual. The plant manager approval signature would be replaced with the signature of a procedurally authorized individual who would be a more appropriate authority for approval of the activity.

*Basis for proposed no significant hazards consideration determination:* As required by Section 50.91(a) of Title 10 of the Code of Federal Regulations (10 CFR), the licensee has provided its analysis of the issue of no significant hazards consideration which is presented below:

1. The proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed change to replace the plant manager's approval with the approval by an

authorized individual is consistent with the requirements of Regulatory Guide 1.33 and American National Standards Institute (ANSI) N18.7-1976/American Nuclear Society (ANS) 3.2. The authorized individuals are management and supervisory personnel who satisfy the requirements of ANSI N18.1-1971. Use of ANSI N18.1-1971 is consistent with the requirements of the existing TS and Updated Safety Analysis Report (USAR). The change is administrative and does not impact or otherwise affect the physical plant.

The proposed change to the License Condition to delete the reporting time frame eliminates duplication of a requirement that is already an integral part of 10 CFR 50.73 which is referenced in the License Condition. The proposed change is administrative and does not impact or otherwise affect the physical plant.

Therefore, the proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. The proposed change would not create the possibility of a new or different kind of accident from any previously evaluated. The proposed administrative changes do not involve any physical modifications to the facility nor add new equipment. The methods of plant operation have not been altered. Therefore, the proposed changes do not create the possibility of a new or different kind of accident from any accident previously evaluated.

3. The proposed changes will not involve a significant reduction in the margin of safety.

The proposed changes are administrative in nature and have no direct impact upon any plant safety analyses. Therefore, the proposed changes do not involve a significant reduction in a margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

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*NRC Section Chief:* Anthony J. Mendiola.

*FirstEnergy Nuclear Operating Company, Docket No. 50-440, Perry Nuclear Power Plant (PNPP), Unit 1, Lake County, Ohio*

*Date of amendment request:* January 30, 2003.

*Description of amendment request:* This license amendment request would modify the existing minimum critical power ratio (MCPR) safety limit contained in Technical Specification (TS) 2.1.1.2. Specifically, the change modifies the MCPR safety limit values, as calculated by Global Nuclear Fuel

(GNF), by decreasing the limit for two recirculation loop operation from 1.10 to 1.07, and decreasing the limit for single recirculation loop operation from 1.11 to 1.08. The change resulted from the core reload analysis performed for the Perry Nuclear Power Plant (PNPP) fuel cycle 10.

*Basis for proposed no significant hazards consideration determination:* As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration which is presented below:

1. The proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

PPNP Updated Safety Analysis Report (USAR) Section 4.2, "Fuel System Design," states the PNPP fuel system design bases are provided in the General Electric Topical Report, NEDE-24011-P-A, "General Electric Standard Application for Reactor Fuel (GESTAR II)." The MCPR Safety Limit is one of the limits used to protect the fuel in accordance with the design basis. The MCPR Safety Limit establishes a margin to the onset of transition boiling. The basis of the MCPR Safety Limit remains the same, ensuring that greater than 99.9% of all fuel rods in the core avoid transition boiling. The methodology used to determine the MCPR Safety Limit values is contained within GESTAR II and is NRC approved. The change does not result in any physical plant modifications or physically affect any plant components. As a result, there is no increase in the probability of occurrence of a previously analyzed accident.

The fundamental sequences of accidents and transients have not been altered. The Safety Limit MCPR is established to avoid fuel damage in response to anticipated operational occurrences. Compliance with a MCPR Safety Limit greater than or equal to the calculated value will ensure that less than 0.1% of the fuel rods will experience boiling transition. This in turn ensures fuel damage does not occur following transitions due to excessive thermal stresses on the fuel cladding. The MCPR Operating Limits are set higher (*i.e.*, more conservative) than the Safety Limit such that potentially limiting plant transients prevent the MCPR from decreasing below the MCPR Safety Limit during the transient. Therefore, there is no impact on any limiting USAR Appendix 15B transients. The radiological consequences remain the same as previously stated in the USAR. Therefore, the consequences of an accident do not increase over previous evaluations in the USAR.

2. The proposed change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

The MCPR Safety Limit basis is preserved, which is to ensure that transition boiling does not occur in at least 99.9% of the fuel rods in the core as a result of the postulated limiting transient. The values are calculated in accordance with GESTAR II. The GESTAR II analyses have been accepted by the NRC.