

November 12, 2002, and January 8 and January 29, 2003.

**Brief description of amendments:** The amendments revised the Technical Specifications to allow a one-time change in the Appendix J, Type A containment integrated leakage rate test interval from the currently required 10-year interval to a test interval of 15 years.

**Date of issuance:** March 12, 2003.

**Effective date:** As of the date of issuance and shall be implemented within 30 days from the date of issuance.

**Amendment Nos.:** 211/192.

**Facility Operating License Nos. NPF-9 and NPF-17:** Amendments revised the Technical Specifications.

**Date of initial notice in *Federal Register*:** July 9, 2002 (67 FR 45563).

The supplements dated September 25 and November 12, 2002, and January 8 and January 29, 2003, provided clarifying information that did not change the scope of the May 29, 2002, application or the initial proposed no significant hazards consideration determination.

The Commission's related evaluation of the amendments is contained in a Safety Evaluation dated March 12, 2003.

**No significant hazards consideration comments received:** No.

*Entergy Gulf States, Inc., and Entergy Operations, Inc., Docket No. 50-458, River Bend Station, Unit 1, West Feliciana Parish, Louisiana*

**Date of amendment request:** April 24, 2002, as supplemented by letters dated July 18, December 18 and 20, 2002, and February 19, 2003.

**Brief description of amendment:** The amendment reflects a full-scope implementation of the alternative source term, as described in Regulatory Guide 1.183, "Alternative Radiological Source Terms for Evaluating Design Basis Accidents at Nuclear Power Reactors," pursuant to 10 CFR 50.67, "Accident source term."

**Date of issuance:** March 14, 2003.

**Effective date:** As of the date of issuance and shall be implemented 60 days from the date of issuance.

**Amendment No.:** 132.

**Facility Operating License No. NPF-47:** The amendment revised the Technical Specifications.

**Date of initial notice in *Federal Register*:** June 11, 2002 (67 FR 40021).

The July 18, December 18 and 20, 2002, and February 19, 2003, supplemental letters provided clarifying information that did not change the scope of the original **Federal Register** notice or the original no significant hazards consideration determination.

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated March 14, 2003.

**No significant hazards consideration comments received:** No.

*Entergy Gulf States, Inc., and Entergy Operations, Inc., Docket No. 50-458, River Bend Station, Unit 1, West Feliciana Parish, Louisiana*

**Date of amendment request:** May 14, 2002, as supplemented by letters dated February 12 and 28, 2003.

**Brief description of amendment:** The amendment modifies the surveillance requirements (SRs) pertaining to the testing of the Division 3 standby emergency diesel generator (EDG). The change allows performance of some required surveillance tests for the Division 3 EDG during any mode of plant operation (previously allowed only in Modes 4 (Cold Shutdown) and 5 (Refueling)).

**Date of issuance:** March 14, 2003.

**Effective date:** As of the date of issuance and shall be implemented 30 days from the date of issuance.

**Amendment No.:** 133.

**Facility Operating License No. NPF-47:** The amendment revised the Technical Specifications and Surveillance Requirements.

**Date of initial notice in *Federal Register*:** June 25, 2002 (67 FR 42824).

The February 12, 2003, supplemental letter provided clarifying information and the February 28, 2003, supplemental letter withdrew the requested change to the Note associated with SR 3.8.1.8. The supplemental letters did not change the scope of the original **Federal Register** notice or the original no significant hazards consideration determination.

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated March 14, 2003.

**No significant hazards consideration comments received:** No.

*Entergy Nuclear Operations, Inc., Docket No. 50-286, Indian Point Nuclear Generating Unit No. 3, Westchester County, New York*

**Date of application for amendment:** June 5, 2002, as supplemented on January 9 and March 4, 2003.

**Brief description of amendment:** The amendment revises the Technical Specifications (TSs) to implement the alternate source term methodology for the fuel-handling accident analysis. Specifically, the amendment revises TS 3.9.3, "Containment Penetrations," to: (1) Permit the equipment closure hatch opening and the personnel airlock doors to be capable of being closed during movement of irradiated fuel, (2) allow

use of administrative controls for unisolating containment penetrations during movement of irradiated fuel, (3) delete the containment purge and containment pressure relief requirements and associated surveillances with the reactor subcritical for less than 550 hours, and (4) eliminate the TS applicability "during core alterations." In this regard, the amendment adopts TS Task Force (TSTF) Standard TS Change Travelers TSTF-68, "Containment Personnel Airlock Doors Open During Fuel Movement," TSTF-312, "Administratively Control Containment Penetrations," and, in part, TSTF-51, "Revise Containment Requirements During Handling Irradiated Fuel and Core Alterations." The amendment also revises the Applicability Statements for Limiting Condition for Operation (LCO) 3.3.8 for the fuel storage building emergency ventilation system (FSBEVS) actuation instrumentation and LCO 3.7.13 for the FSBEVS to also add the term "recently" before "irradiated fuel assemblies." In addition, the LCO Required Action would likewise be modified to add the term "recently" to now require the suspension of movement of recently irradiated fuel in the FSB.

**Date of issuance:** March 17, 2003.

**Effective date:** March 17, 2003.

**Amendment No.:** 215.

**Facility Operating License No. DPR-64:** Amendment revised the Technical Specifications.

**Date of initial notice in *Federal Register*:** July 9, 2002 (67 FR 45567).

The January 9 and March 4 letters provided clarifying information that did not expand the scope of the proposed amendment or change the initial proposed no significant hazards consideration determination.

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated March 17, 2003.

**No significant hazards consideration comments received:** No.

*Entergy Nuclear Operations, Inc., Docket No. 50-293, Pilgrim Nuclear Power Station, Plymouth County, Massachusetts*

**Date of application for amendment:** August 16, 2002.

**Brief description of amendment:** The amendment relocates certain Control Rod Block functions from Technical Specifications 3/4.2.C, "Control Rod Block Actuation," Tables 3.2.C.1, 3.2.C-2, and 4.2.C to the Updated Final Safety Analysis Report.

**Date of issuance:** March 17, 2003.