

Basis for proposed no significant hazards consideration determination: As required by Title 10 of the Code of Federal Regulations (10 CFR) 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration. The NRC staff has reviewed the licensee's analysis against the standards of 10 CFR 50.92(c). The NRC staff's review is presented below:

1. Involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed change involves allowing VY to operate in an expanded operating domain. Physical changes provide for enhanced instrument performance or were the result of safety analyses that support mitigation of design bases accidents. There are no changes to radioactive source terms or release pathways. The proposed change does not result in any significant change in the availability of logic systems or safety-related systems themselves. Required protective functions will be maintained. The proposed change does not degrade plant design, operation, or the performance of any safety system assumed to function in the accident analysis.

Therefore, the proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. Create the possibility for a new or different kind of accident from any previously evaluated.

The proposed change, which allows VY to operate in an expanded operating domain, does not introduce any new accidents or failure mechanisms because the change and the effects on existing structures, systems and components have been evaluated and found to not have any adverse effects. The proposed change will not substantially impose new requirements or eliminate any existing requirements.

Therefore, the proposed change does not create the possibility of a new or different kind of accident than those previously evaluated.

3. Involve a significant reduction in a margin of safety.

The proposed change, which allows VY to operate in an expanded operating domain, does not alter the manner in which safety limits, limiting safety system settings, or limiting conditions for operation are determined. There is no impact on the conclusions of any safety analysis. The proposed change does not involve any increase in calculated off-site dose consequences. The performance of equipment will not be significantly affected.

Therefore, there is no significant reduction in the margin of safety as a result of this proposed change.

Based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

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NRC Section Chief: James W. Clifford.

Exelon Generation Company, LLC, Docket Nos. 50-237 and 50-249, Dresden Nuclear Power Station, Units 2 and 3, Grundy County, Illinois

Date of amendment request: January 31, 2003.

Description of amendment request: The proposed amendments would change the Technical Specifications (TS) allowable values (AVs) for isolation condenser system isolation Function 4.a, Steam Flow-High, and Function 4.b, Return Flow-High.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The proposed TS changes do not involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed TS changes support the replacement of a differential pressure switch with a functionally equivalent differential pressure switch. Since there are no functional changes and no change in analytical limits, there is no significant increase in the probability or consequences of an accident previously evaluated.

Additionally, these changes will not increase the consequences of an accident previously evaluated because the proposed changes do not adversely impact structures, systems, or components. Furthermore, there will be no change in the types or significant increase in the amounts of any effluents released offsite as a result of the proposed change.

In summary, the proposed changes do not involve a significant increase in the probability of consequences of an accident previously evaluated.

2. The proposed TS changes do not create the possibility of a new or different kind of accident from any accident previously evaluated.

The change does not adversely impact the manner in which the instrument will operate under normal and abnormal operating conditions. Therefore, these changes provide an equivalent level of safety and will not create the possibility of a new or different kind of accident from any accident previously evaluated. The changes in allowed values do not affect the current safety analysis assumptions. Therefore, these changes will not create the possibility of a new or different kind of accident from any accident previously evaluated.

3. The proposed TS changes do not involve a significant reduction in a margin of safety.

The proposed changes do not affect the probability of failure or availability of the affected instrumentation. The revised AVs do not affect the analytical limits assumed in the safety analyses for actuation of

instrumentation. Therefore, the proposed changes do not result in a reduction in the margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the requested amendments involve no significant hazards consideration.

Attorney for licensee: Mr. Edward J. Cullen, Deputy General Counsel, Exelon BSC—Legal, 2301 Market Street, Philadelphia, PA 19101.

NRC Section Chief: Anthony J. Mendiola.

Florida Power Corporation, et al., Docket No. 50-302, Crystal River Unit No. 3 Nuclear Generating Plant, Citrus County, Florida

Date of amendment request: February 17, 2003.

Description of amendment request: The proposed amendment would revise Technical Specification (ITS) 3.6.3 "Containment Isolation Valves," to allow verification by administrative means of isolation devices in high radiation areas, and isolation devices that are locked, sealed or otherwise secured. The specific Conditions and Surveillance Requirements (SR) in ITS 3.6.3 that will be affected are: (1) Condition A—Required Action A.2, (2) Condition B—Required Actions B.1 and B.2, (3) Condition C—Required Action C.2, and (4) SR 3.6.3.3 and SR 3.6.3.4. The licensee stated that the changes are consistent with the NUREG-1430, "Standard Technical Specifications: Babcock and Wilcox Plants," Revision 2, and Standard Technical Specification Task Force (TSTF) Traveler TSTF-440. Associated changes are also proposed for the ITS Bases.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. Does not involve a significant increase in the probability or consequences of an accident previously analyzed.

The proposed License Amendment Request (LAR) will revise the position verification requirements for manual containment isolation devices that are locked, sealed, or otherwise secured in the closed position. The proposed changes will allow the use of administrative controls to verify the position of these types of devices when they are being used to meet the Required Actions of ITS 3.6.3 Condition A, Condition B or Condition C, and will exclude these valves from Surveillance Requirement (SR) 3.6.3.3 and