

accidentally or as a result of a design basis event.

Therefore, this proposed amendment will not involve a significant reduction in a margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

Attorney for licensee: Mr. Alvin Gutterman, Morgan Lewis, 1111 Pennsylvania Avenue NW., Washington, DC 20004.

NRC Section Chief: L. Raghavan.

Nuclear Management Company, LLC, Docket No. 50-263, Monticello Nuclear Generating Plant, Wright County, Minnesota

Date of amendment request: January 29, 2003.

Description of amendment request: The proposed amendment would change the drywell leakage and sump monitoring detection section of the Technical Specifications (TSs). These proposed changes clarify the definitions and restructure the coolant leakage section of the TSs and revise unidentified leakage and total leakage requirements. The revisions add a TS Limiting Condition for Operation for leakage-detection instrumentation being inoperable. This request supercedes the Nuclear Management Company's license amendment request of October 8, 2002, as supplemented November 8, 2002, which was previously noticed in the **Federal Register** on October 17, 2002 (67 FR 64144).

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The proposed amendment will not involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed Technical Specification changes do not introduce new equipment or new equipment operating modes, nor do the proposed changes alter existing system relationships. Additionally, the proposed changes do not affect any accident previously evaluated in the Monticello Updated Safety Analysis Report (USAR). The changes simply redefine the parameters for evaluation of leakage in the drywell. The evaluation criteria for drywell leakage have been refocused into the areas that are most susceptible to IGSCC [intergranular stress corrosion cracking]. Consequently, the probability of an accident previously evaluated is not significantly increased.

The equipment referenced in the proposed changes is still required to monitor the reactor coolant system operational leakage to ensure appropriate action is taken before the integrity of the reactor coolant pressure boundary is impaired. As a result, operation of the facility with the proposed changes will continue to meet the licensing basis and applicable guidelines. As such, the consequences of any accident previously evaluated are not significantly affected.

Therefore, the proposed amendment will not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. The proposed amendment will not create the possibility of a new or different kind of accident from any accident previously analyzed.

The proposed changes do not involve physical alterations of the plant; no new or different type of equipment will be installed; nor are there significant changes in the methods governing normal plant operation. The changes simply redefine the parameters for evaluation of leakage in the drywell. The evaluation criteria for drywell leakage have been refocused into the areas that are most susceptible to IGSCC. Additionally, the changes do not create any new failure mechanisms, malfunctions, or accident initiators not already considered in the design and licensing bases.

Therefore, the proposed amendment will not create the possibility of a new or different kind of accident from any accident previously analyzed.

3. The proposed amendment will not involve a significant reduction in the margin of safety.

The proposed amendment redefines the parameters for evaluation of leakage in the drywell. There are no physical alterations of the plant; no new or different type of equipment will be installed; nor are there significant changes in the methods governing normal plant operation. Additionally, the proposed changes do not exceed or alter a design basis or safety limit as established in the Monticello licensing basis.

Therefore, these proposed changes will not involve a significant reduction in the margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

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NRC Section Chief: L. Raghavan.

Nuclear Management Company, LLC, Docket Nos. 50-282 and 50-306, Prairie Island Nuclear Generating Plant, Units 1 and 2, Goodhue County, Minnesota

Date of amendment request: February 11, 2003.

Description of amendment request:

The proposed amendments would revise technical specification (TS) 5.5.9, "Ventilation Filter Testing Program (VFTP)" by (1) incorporating filter test face velocity limits for the control room special ventilation system, auxiliary building special ventilation system, spent fuel pool special and inservice purge ventilation system, and shield building ventilation system; and (2) making editorial changes. The proposed amendments would also delete the additional conditions in Appendix B of the Operating Licenses which require the licensee to complete an evaluation of the maximum test face velocity for the ventilation systems in TS 5.5.9. The additional conditions also require the licensee to submit a license amendment request for a TS amendment to specify the maximum test face velocity if the maximum actual face velocity is the greater than 110 percent of 40 fpm. Additionally, the proposed amendments would revise the penetration and system bypass limit from 0.05 percent to 0.5 percent for the ventilation systems.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

Revision of the Allowable Filtration Penetration and System Bypass

1. The proposed amendment will not involve a significant increase in the probability or consequences of an accident previously evaluated.

This license amendment request proposes to increase the penetration and system bypass limit for the control room special ventilation system, auxiliary building special ventilation system, spent fuel pool special and inservice purge ventilation system and shield building ventilation system from 0.05% to 0.5%. These ventilation systems are included in the plant design to mitigate accident consequences and are not assumed accident initiators, thus, this change does not involve a significant increase in the probability of an accident. This change will assure that the subject ventilation systems will perform within their intended design ranges thus, this change assures that the consequences of an accident are not increased.

2. The proposed amendment will not create the possibility of a new or different kind of accident from any accident previously analyzed.

This proposed change does not alter the design, function, or operation of any plant component and does not install any new or different equipment. The malfunction of safety related equipment, assumed to be operable in the accident analyses, would not be caused as a result of the proposed Technical Specification change. No new