

Specification change takes credit for the National Academy for Nuclear Training accreditation of the licensed operator training program. The Technical Specification requirements for all other plant staff qualifications remain unchanged.

Additionally, the proposed Technical Specification change does not affect plant design, hardware, system operation, or procedures. Therefore, the proposed change does not create the possibility of a new or different kind of accident from any previously evaluated.

3. Does the proposed change involve a significant reduction in a margin of safety?

*Response:* No.

The proposed Technical Specification change is an administrative change to clarify the current requirements applicable to licensed operator qualifications and the licensed operator training program. With this change the Technical Specifications continue to be consistent with the requirements of 10 CFR [Part] 55. The Technical Specification qualification requirements for all other plant staff remain unchanged.

Licensed operator qualifications and training can have an indirect impact on a margin of safety. However, the NRC considered this impact during the rulemaking process, and by promulgation of the revised 10 CFR [Part] 55, determined that this impact remains acceptable when licensees maintain a licensed operator training program that is accredited and based on a systems approach to training. As noted previously, the Prairie Island Nuclear Generating Plant licensed operator training program is accredited by the National Academy for Nuclear Training and is based on a systems approach to training.

The NRC has concluded, as stated in NUREG-1262, "Answers to Questions at Public Meetings Regarding Implementation of Title 10, Code of Federal Regulations, Part 55 on Operators' Licenses," that the standards and guidelines applied by the Institute for Nuclear Power Operations' National Academy for Nuclear Training in their training accreditation program are equivalent to those put forth or endorsed by the NRC. As a result, maintaining a National Academy for Nuclear Training accredited, systems approach based licensed operator training program is equivalent to maintaining an NRC approved licensed operator training program which conforms with applicable NRC Regulatory Guides or NRC endorsed industry standards. The margin of safety is maintained by virtue of maintaining the National Academy for Nuclear Training accredited licensed operator training program.

In addition, the NRC published NRC Regulatory Issue Summary 2001-01, "Eligibility of Operator License Applicants," dated January 18, 2001, "to familiarize addressees with the NRC's current guidelines for the qualification and training of reactor operator (RO) and senior operator (SO) license applicants." This document again acknowledges that the Institute for Nuclear Power Operations' National Academy for Nuclear Training guidelines for education and experience, outline acceptable methods for implementing the NRC's regulations in this area.

Therefore, the proposed changes do not involve a significant reduction in a margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment requests involve no significant hazards consideration.

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*Pacific Gas and Electric Company, Docket Nos. 50-275 and 50-323, Diablo Canyon Nuclear Power Plant, Unit Nos. 1 and 2, San Luis Obispo County, California*

*Date of amendment requests:* February 6, 2003.

*Description of amendment requests:* The proposed license amendments would revise Surveillance Requirements (SRs) 3.3.1.2 and 3.3.1.3 of TS 3.3.1, "Reactor Trip System Instrumentation," of the Diablo Canyon Technical Specifications. The change to SR 3.3.1.2 is consistent with NRC-approved Industry/Technical Specifications Task Force (TSTF) Standard Technical Specification Change Traveler, TSTF-371. The change to SR 3.3.1.3 is editorial in nature.

*Basis for proposed no significant hazards consideration determination:* As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed change to Technical Specifications (TS) Surveillance Requirement (SR) 3.3.1.2 and SR 3.3.1.3 is consistent with the NRC approved Industry/Technical Specifications Task Force Standard Technical Specification Change Traveler, TSTF-371, and NUREG-1431, "Standard Technical Specifications, Westinghouse Plants," Revision 2.

Overall protection system performance will remain within the bounds of the previously performed accident analyses since there are no hardware changes. The reactor trip system (RTS) instrumentation will be unaffected. Protection systems will continue to function in a manner consistent with the plant design basis. All design, material, and construction standards that were applicable prior to the request are maintained.

The probability and consequences of accidents previously evaluated in the Updated Final Safety Analysis Report (UFSAR) are not adversely affected because

the change to the nuclear instrumentation system (NIS) power range channel daily surveillance assures the conservative response of the channel even at part-power levels.

The proposed change modifies the NIS power range channel daily surveillance requirement to help assure the NIS power range functions are tested in a manner consistent with the safety analysis and licensing basis.

The proposed change will not affect the probability of any event initiators. There will be no degradation in the performance of, or an increase in the number of challenges imposed on, safety-related equipment assumed to function during an accident situation. There will be no change to normal plant operating parameters or accident mitigation performance.

The proposed change will not alter any assumptions or change any mitigation actions in the radiological consequence evaluations in the USAR.

Therefore, the proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. The proposed change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

There is no hardware change or change in the method by which any safety-related plant system performs its safety function. This change will not affect the normal method of plant operation or change any operating parameters. No performance requirements or response time limits will be affected. The NIS power range high trip setpoint adjustment requirements, prior to adjusting indicated power in a decreasing power direction, will ensure the reactor power level is consistent with assumptions made in the safety analysis and licensing basis.

No new accident scenarios, transient precursors, failure mechanisms, or limiting single failures are introduced as a result of the proposed change. There will be no adverse effect or challenges imposed on any safety-related system as a result of the change.

This amendment does not alter the design or performance of the Eagle 21 System, NIS, or Solid State Protection System used in the plant protection systems.

Therefore, the proposed change does not create the possibility of a new or different kind of accident from any previously evaluated.

3. The proposed change does not involve a significant reduction in a margin of safety.

The proposed change requires a revision to the criteria for implementation of NIS power range channel adjustments based on secondary power calorimetric calculations; however, the change does not eliminate any RTS surveillances or alter the frequency of surveillances required by the Technical Specifications. The revision to the criteria for implementation of the daily surveillance will have a conservative effect on the performance of the NIS power range channels, particularly at part-power conditions. The nominal trip setpoints specified in the Technical Specification Bases and the safety analysis