

consideration, which is presented below: Pursuant to 10 CFR 50.91, Duke Power Company (Duke) has made the determination that this amendment request involves a No Significant Hazards Consideration by applying the standards established by the NRC regulations in 10 CFR 50.92. This ensures that operation of the facility in accordance with the proposed amendment would not:

1. Involve a significant increase in the probability or consequences of an accident previously evaluated: The proposed LAR [Licence Amendment Request] modifies the Technical Specifications [(TS)] to incorporate new TS requirements associated with the new Low Pressure Injection (LPI) System configuration and eliminate TS requirements associated with the old LPI configuration. The proposed LAR also modifies the licensing basis to adopt Standard Review Plan (SRP) 3.6.2 Branch Technical Position (BTP) MEB 3-1 requirements for selected portions of LPI piping and to credit Leak-Before-Break (LBB) to allow the dynamic effects associated with postulated pipe rupture of selected portions of the LPI/Core Flood (CF) piping to be excluded from the design basis. The proposed design allowances for these selected portions of piping continue to allow the LPI system design to meet GDC [General Design Criterion] 4 requirements related to environmental and dynamic effects. The proposed LAR will continue to ensure that ONS [Oconee Nuclear Station] can meet design basis requirements associated with the LPI safety function. The LPI System provides a means for delivering a large volume of borated water to the reactor core following postulated large pipe breaks in the Reactor Coolant System. The planned modification adds a passive crossover connection between the two LPI injection lines inside containment, along with necessary check valves and flow orifices that will eliminate the need for time-critical operator actions to manually open the LPI discharge header outside containment. The new components will have the same pressure, seismic, and quality group qualifications as the existing components in the LPI system. The addition of the crossover line will enhance the ability of the control room operator to mitigate the consequences of specific events for which LPI is credited. Therefore, the proposed LAR does not involve a significant increase in the probability or consequences of an accident previously evaluated.

The LPI system is also relied on to cool the reactor core during unit shutdown. Hydraulic analyses have

demonstrated that adequate LPI flow is available for normal shutdown cooling with the new LPI piping configuration.

2. Create the possibility of a new or different kind of accident from any kind of accident previously evaluated: The proposed LAR modifies the Technical Specification to incorporate new TS requirements associated with the new LPI System configuration and eliminate TS requirements associated with the old LPI System configuration. The proposed LAR also modifies the licensing basis to adopt MEB 3-1 requirement for selected portions of LPI piping and to credit LBB to allow the dynamic effects associated with postulated pipe rupture of selected portions of the LPI/Core Flood (CF) piping to be excluded from the design basis. The proposed design allowances for these selected portions of piping continue to allow the LPI system design to meet GDC 4 requirements related to environmental and dynamic effects. The LPI and Core Flood systems affected by implementing the proposed changes to the TS are not assumed to initiate design basis accidents. The systems affected by the changes are used to mitigate the consequences of an accident that has already occurred. The proposed TS and licensing basis changes do not affect the mitigating function of these systems. Consequently, these changes do not alter the nature of events postulated in the Safety Analysis Report nor do they introduce any unique precursor mechanisms. Therefore, the proposed amendment will not create the possibility of a new or different kind of accident from any accident previously evaluated.

3. Involve a significant reduction in a margin of safety.

The proposed TS and licensing basis changes do not unfavorably affect any plant safety limits, set points, or design parameters. The changes also do not unfavorably affect the fuel, fuel cladding, RCS, or containment integrity. Therefore, the proposed TS and licensing basis changes, which adds TS requirements and adopts new design allowances associated with the passive LPI cross connect modification, do not involve a significant reduction in the margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

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Entergy Gulf States, Inc., and Entergy Operations, Inc., Docket No. 50-458, River Bend Station, Unit 1, West Feliciana Parish, Louisiana

Date of amendment request: March 19, 2003.

Description of amendment request:

The proposed amendment deletes requirements from the technical specifications (TS) and other elements of the licensing bases to maintain a Post Accident Sampling System (PASS). Licensees were generally required to implement PASS upgrades as described in NUREG-0737, "Clarification of TMI [Three Mile Island Nuclear Station] Action Plan Requirements," and Regulatory Guide 1.97, "Instrumentation for Light-Water-Cooled Nuclear Power Plants to Assess Plant and Environs Conditions During and Following an Accident." Implementation of these upgrades was an outcome of the lessons learned from the accident that occurred at TMI, Unit 2 (TMI-2). Requirements related to PASS were imposed by Order for many facilities and were added to or included in the TS for nuclear power reactors currently licensed to operate. Lessons learned and improvements implemented over the last 20 years have shown that the information obtained from PASS can be readily obtained through other means or is of little use in the assessment and mitigation of accident conditions.

The changes are based on NRC-approved Technical Specification Task Force (TSTF) Standard Technical Specification Change Traveler, TSTF-413, "Elimination of Requirements for a Post Accident Sampling System (PASS)." The U.S. Nuclear Regulatory Commission (NRC) staff issued a notice of opportunity for comment in the **Federal Register** on December 27, 2001 (66 FR 66949), on possible amendments concerning TSTF-413, including a model safety evaluation and model no significant hazards consideration (NSHC) determination, using the consolidated line item improvement process. The NRC staff subsequently issued a notice of availability of the models for referencing in license amendment applications in the **Federal Register** on March 20, 2002 (67 FR 13027). The licensee affirmed the applicability of the following NSHC determination in its application dated March 19, 2003.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), an