

Bases, is not significantly reduced in any of the Point Beach licensing basis accident analyses based on the subject changes to safety analyses input parameter values. There are no new or significant changes to the initial conditions contributing to accident severity or consequences. Since the analyses in the accompanying submittals [March 27, 2003, application and WCAP-15432] demonstrate that all applicable acceptance criteria continue to be met, the subject operating conditions will not involve a significant reduction in a margin of safety at Point Beach.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

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Nuclear Management Company, LLC, Docket Nos. 50-282 and 50-306, Prairie Island Nuclear Generating Plant, Units 1 and 2, Goodhue County, Minnesota

Date of amendment request: March 25, 2003.

Description of amendment request: The proposed amendments would revise Technical Specification (TS) 3.2.1 and TS 3.2.3 for implementation of relaxed axial offset control of the reactor cores, relocate selected operating parameters from TS 2.0 and TS 3.3.1 to the Core Operating Limits Report (COLR), revise the Pressurizer Pressure-Low Allowable Value, and revise the appropriate references in TS 5.6.5 to the NRC-approved methodologies which support relocation of operating parameters to the COLR.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

Group 1—Implementation of Relaxed Axial Offset Control

A. TS 3.2.1, Heat Flux Hot Channel Factor— $F_Q(Z)$ and Bases: Modification of Required Actions and Completion Time if $FW_Q(Z)$ is not within its limit and update Bases.

B. TS 3.2.3, Axial Flux Difference (AFD) and Bases: Modification of Limiting Conditions for Operation, Actions and Surveillance Requirements and revision of the Bases.

This license amendment request proposes to revise the Technical Specifications to implement the relaxed axial offset control methodology to address the heat flux hot channel factor and axial flux difference limits.

1. The proposed amendment will not involve a significant increase in the probability or consequences of an accident previously evaluated.

This license amendment request proposes to revise the Technical Specifications to implement the relaxed axial offset control methodology to address the heat flux hot channel factor and axial flux difference limits. The revised Technical Specifications and parameter changes associated with relaxed axial offset control assure that the limiting safety analysis inputs (such as, heat flux hot channel factor and axial flux difference limits) are not exceeded. The bounding power distribution transient factor values, $W(Z)$, and the axial flux difference limits that are documented in the Core Operating Limits Report will be determined by NRC approved analytical methods and will be validated as part of the cycle specific reload evaluation process.

Heat flux hot channel factors and axial flux difference limits are not assumed accident initiators. Therefore, the relaxed axial offset control related Technical Specification changes do not involve a significant increase in the probability of an accident.

Likewise, operation of the plant within the proposed Technical Specification controls and limits assures that safety analysis assumptions are met, thus, if an accident were to occur, the consequences would continue to be bounded by the accident analyses. Therefore, the relaxed axial offset control related technical specification changes do not involve a significant increase in the consequences of an accident.

The relaxed axial offset control related technical specification changes do not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. The proposed amendment will not create the possibility of a new or different kind of accident from any accident previously analyzed.

This proposed change does not involve a physical alteration of the plant; that is, no new or different type of equipment will be installed. This proposed change does not introduce any new mode of plant operation or change the methods governing normal plant operation. No new failure mode has been created and no new equipment performance burdens are imposed. Therefore the possibility of a new or different kind of accident from those previously analyzed has not been created.

3. The proposed amendment will not involve a significant reduction in the margin of safety.

This license amendment request proposes to revise the Technical Specifications to implement the relaxed axial offset control methodology to address the heat flux hot channel factor and axial flux difference limits. The supporting Technical Specification limits are defined by NRC approved analytical methods which are

performed to conservatively bound the operating conditions defined by the Technical Specifications and to demonstrate meeting the regulatory acceptance limits. The heat flux hot channel factor licensed safety margins are maintained. The heat flux hot channel factor conforms to plant design bases and limits actual plant operation within analyzed and licensed boundaries. The relaxed axial offset control methodology has been demonstrated to ensure that core heat flux hot channel factors will remain below accident analysis limits. The margin of safety provided by the analyses in accordance with the acceptance limits is maintained and not reduced. Thus, the implementation of relaxed axial offset control at Prairie Island does not involve a significant reduction in a margin of safety.

Group 2—Relocation of Technical Specifications Safety Limits Figure and Overtemperature Delta-T and Overpower Delta-T Parameter Values to the Core Operating Limits Report, and Miscellaneous Administrative Changes

A. TS 2.1.1, "Reactor Core SLs [Safety Limits]" and Bases: Relocate the safety limits Figure to the Core Operating Limits Report, update TS 2.1.1 and Bases.

B. TS 3.3.1, Table 3.3.1-1 (Pages 2, 7 and 8), "Reactor Trip System Instrumentation", Overpower Delta-T Trip Function, and Overtemperature Delta-T and Overpower Delta-T parameter values: Delete SR [Surveillance Requirement] 3.3.1.3, SR 3.3.1.6, and remove f(DI) from Overpower Delta-T Trip Function, relocate overtemperature delta-T and overpower delta-T parameter values and revise the Bases.

C. TS 5.6.5, Core Operating Limits Report (COLR): Additions to document Technical Specifications with limits in the Core Operating Limits Report and the analytical methods used to determine the values for relocated safety limits and overtemperature delta-T and overpower delta-T parameters and miscellaneous administrative changes.

This license amendment request proposes to relocate the safety limits and overtemperature delta-T and overpower delta-T parameter values to the Core Operating Limits Report. Relocation of these limits and parameter values to the Core Operating Limits Report allows them to be changed under licensee controls. This license amendment also proposes to include, in the Technical Specifications administrative controls section, the appropriate references to the NRC approved methodologies which will be used to determine the safety limits and overtemperature delta-T and overpower delta-T parameter values. These changes are acceptable because the values used to operate the Prairie Island plant will be determined using NRC approved methods and these changes are consistent with the guidance of the industry standard Technical Specifications, NUREG-1431, Revision 2, "Standard Technical Specifications Westinghouse Plants". This license amendment request also proposes to delete references to an NRC Safety Evaluation and make some editorial corrections in the Technical Specifications administrative controls section. These changes are