

Creek Generating Station (HCGS) reactor vessel material surveillance program required by Appendix H to Title 10 of the Code of Federal Regulations (10 CFR) part 50. This change would incorporate the Boiling Water Reactor Vessel and Internals Project Integrated Surveillance Program (ISP) into the HCGS licensing basis.

*Basis for proposed no significant hazards consideration determination:* As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. Does the change involve a significant increase in the probability or consequences of an accident previously analyzed?

*Response:* No.

The proposed change implements an integrated surveillance program that has been evaluated by the NRC staff as meeting the requirements of paragraph III.C of Appendix H to 10 CFR 50. Consequently, the proposed change does not significantly increase the probability of any accident previously evaluated. The proposed change provides the same assurance of RPV [reactor pressure vessel] integrity. As a result, the consequences of any accident previously evaluated are not significantly increased.

Therefore, this proposed amendment does not involve a significant increase in the probability of occurrence or consequences of an accident previously analyzed.

2. Does the change create the possibility of a new or different kind of accident from any accident previously analyzed?

*Response:* No.

The proposed change revises the HCGS licensing basis to reflect participation in the ISP. The proposed change does not involve a modification of the design of plant structures, systems or components (SSC). Also, the proposed change will not degrade the reliability of SSCs important to safety since protective features will not be deleted or modified. The proposed change will not impact the manner in which the plant is normally operated. The proposed change maintains an equivalent level of RPV material surveillance and does not introduce any new accident initiators. Therefore, this proposed amendment does not create the possibility of a new or different kind of accident from any previously analyzed.

3. Does the change involve a significant reduction in the margin of safety?

*Response:* No.

The proposed change has been evaluated as providing an acceptable alternative to the plant-specific RPV material surveillance program that meets the requirements of the regulations for RPV material surveillance. Therefore, these changes do not involve a significant reduction in margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the

amendment request involves no significant hazards consideration.

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*NRC Section Chief:* James W. Clifford.

*PSEG Nuclear LLC, Docket Nos. 50–272 and 50–311, Salem Nuclear Generating Station, Unit Nos. 1 and 2, Salem County, New Jersey*

*Date of amendment request:* September 20, 2002, as revised on February 14, 2003. This notice supercedes a previous notice (67 FR 75884) published on December 10, 2002, which was based on the licensee's application dated September 20, 2002.

*Description of amendment request:* The proposed amendment will: (1) Add a new limiting condition for operation (LCO) for spent fuel pool (SFP) boron concentration; (2) relocate requirements from Technical Specification (TS) Section 5.0, "Design Features," to a new LCO in TS Section 3/4.7; and (3) revise existing TS 3/4.9.1 for refueling operations by relocating requirements for boron concentration to the Core Operating Limits Report (COLR) described in TS 6.9.1.9. The licensee also proposed related changes to the TS Bases. By letter dated February 14, 2003, PSEG revised its request, including lowering the minimum SFP boron concentration from 2300 parts per million (ppm) to 800 ppm.

Therefore, this notice supercedes a previous notice published on December 10, 2002, to reflect this change.

*Basis for proposed no significant hazards consideration determination:* As required by Title 10 of the Code of Federal Regulations (10 CFR), Section 50.91(a), the licensee has provided a revised analysis of the issue of no significant hazards consideration. The NRC staff has reviewed the licensee's analysis against the standards of 10 CFR 50.92(c). The NRC staff's review is presented below:

1. Does the proposed change involve a significant increase in the probability or consequences of an accident previously evaluated?

The licensee proposed to change the Salem Nuclear Generating Station (Salem) TSs by: (1) adding a new LCO for SFP boron concentration; (2) relocating requirements from TS Section 5.0, "Design Features," to a new LCO in TS Section 3/4.7; and (3) revising existing TS 3/4.9.1 for refueling operations by relocating requirements for boron concentration to the COLR. These changes are consistent with applicable LCOs in NUREG-1431, Revision 2, "Improved Standard Technical Specifications, Westinghouse

Plants," and will continue to provide administrative controls to ensure that a proper boron concentration is maintained in accordance with Salem's accident analyses. Because there are no changes to any of the input assumptions associated with postulated accidents involving refueling operations and the SFP, the proposed amendment does not involve a significant increase in the probability of occurrence or consequences of an accident previously analyzed.

2. Does the proposed change create the possibility of a new or different kind of accident from any accident previously analyzed?

Adding new LCOs for boron concentration in the SFP and relocating boron concentration requirements to the COLR will not change the conduct of operations in the SFP, refueling cavity and fuel transfer tube at Salem. Therefore, because plant operations will not change, the proposed amendment does not create the possibility of a new or different kind of accident from any previously analyzed.

3. Does the proposed change involve a significant reduction in the margin of safety?

Refueling operations and SFP boron concentration limits will be based on approved methodologies and accident analyses that are unchanged as a result of the proposed TS amendments. Therefore, because existing margins of safety will be maintained, the proposed change does not involve a significant reduction in the margin of safety.

Based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

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*NRC Section Chief:* James W. Clifford.

*Pacific Gas and Electric Company, Docket Nos. 50–275 and 50–323, Diablo Canyon Nuclear Power Plant, Unit Nos. 1 and 2, San Luis Obispo County, California*

*Date of amendment requests:* February 28, 2003.

*Description of amendment requests:* The proposed license amendments would revise Action A of Technical Specification (TS) 3.5.2, "ECCS—Operating," to change the completion time for restoring centrifugal charging pump (CCP) 1–1 to operable status during Diablo Canyon Power Plant (DCPP) Unit 1 Cycle 12, from 72 hours to 7 days. The 72-hour allowed completion time is not sufficient to