

accomplish such emergent repairs on an inoperable CCP. This license amendment request also removes a similar one-time change for DCCP Unit 2 CCP 2-1 which has expired.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

The emergency core cooling system (ECCS) and the centrifugal charging pumps (CCPs) are designed to respond to mitigate the consequences of an accident. They are not an accident initiator, and as such cannot increase the probability of an accident.

The loss of both CCPs, due to an inoperable CCP 1-1 and a single failure of CCP 1-2, could increase the consequences of an accident. A probabilistic risk assessment was performed to evaluate the increased consequences. The worst case risk increment due to the increased completion time for CCP 1-1 and the maximum allowed results in only a small quantitative impact on plant risk.

Allowing 7 days to complete the seal replacement and post-maintenance testing of CCP 1-1 is acceptable since the ECCS system remains capable of performing its intended function of providing at least the minimum flow assumed in the accident analyses. During the extended maintenance and test period, appropriate compensatory measures will be implemented to restrict high risk activity. The consequences of accidents, which rely on the ECCS system, will not be significantly affected.

Therefore, the proposed changes will not result in a significant increase in the probability or consequences of an accident previously evaluated.

2. The proposed change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

There are no new failure modes or mechanisms created due to plant operation for an extended period to perform repairs and post-maintenance testing of CCP 1-1. Extended operation with an inoperable CCP does not involve any modification in the operational limits or physical design of the systems. There are no new accident precursors generated due to the extended allowed completion time.

Therefore, the proposed changes do not create the possibility of a new or different kind of accident from any accident previously evaluated.

3. The proposed change does not involve a significant reduction in a margin of safety.

Plant operation for seven days with an inoperable CCP 1-1 does not adversely affect the margin of safety. During the extended allowable completion time the ECCS system maintains the ability to perform its safety function of providing at least the minimum

flow assumed in the accident analyses. During the extended maintenance and test period, appropriate compensatory measures will be implemented to restrict high-risk activity.

Therefore, the change does not involve a significant reduction in a margin of safety as defined in the basis for any Technical Specification.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment requests involve no significant hazards consideration.

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Previously Published Notices of Consideration of Issuance of Amendments to Facility Operating Licenses, Proposed no Significant Hazards Consideration Determination, and Opportunity for a Hearing

The following notices were previously published as separate individual notices. The notice content was the same as above. They were published as individual notices either because time did not allow the Commission to wait for this biweekly notice or because the action involved exigent circumstances. They are repeated here because the biweekly notice lists all amendments issued or proposed to be issued involving no significant hazards consideration.

For details, see the individual notice in the **Federal Register** on the day and page cited. This notice does not extend the notice period of the original notice.

Florida Power & Light Company, et al. (FPL's), Docket Nos. 50-335, and 50-389, St. Lucie Plant, Unit No. 1, and Unit No. 2, St. Lucie County, Florida

Date of amendment request: October 23, 2002.

Description of amendment request: The proposed license amendments would revise the Technical Specifications to include the design of a new cask pit spent fuel storage rack for each unit to increase the allowable spent fuel wet storage capacity at both units and include the description of Boral™ as the neutron absorbing material used in the new cask pit storage racks. The proposal would also revise the spent fuel pool thermal-hydraulic analyses for core offload times and include a change in FPL's commitments regarding the Unit 2 spent fuel cooling system design basis

described in the Updated Final Safety Analysis Report.

Date of publication of individual notice in the Federal Register: January 28, 2003 (68 FR 4244), as corrected March 31, 2003 (64 FR 15487).

Expiration date of individual notice: February 27, 2003.

Tennessee Valley Authority, Docket No. 50-327, Sequoyah Nuclear Plant, Unit 1, Hamilton County, Tennessee

Date of application for amendments: February 14, 2003.

Description of amendments request: Revise the Updated Final Analysis Report to change the methodology using a through-bolted connection frame that is different than the original design and construction of the steam generator roof compartment.

Date of publication of individual notice in the Federal Register: March 14, 2003 (68 FR 12382).

Expiration date of individual notice: April 14, 2003.

Tennessee Valley Authority, Docket No. 50-327, Sequoyah Nuclear Plant, Unit 1, Hamilton County, Tennessee

Date of application for amendments: March 18, 2003.

Description of amendments request: Revise the Updated Final Analysis Report to provide an alternative methodology using a Bar-Lock mechanical splice in lieu of the Cadweld splice used in the original design and construction of the concrete shield building dome.

Date of publication of individual notice in the Federal Register: March 17, 2003 (68 FR 12718).

Expiration date of individual notice: April 16, 2003.

Notice of Issuance of Amendments to Facility Operating Licenses

During the period since publication of the last biweekly notice, the Commission has issued the following amendments. The Commission has determined for each of these amendments that the application complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment.

Notice of Consideration of Issuance of Amendment to Facility Operating License, Proposed No Significant Hazards Consideration Determination, and Opportunity for A Hearing in connection with these actions was