

2002, and January 14, January 29, January 31, and February 5, 2003; (2) the Commission's Safety Evaluation Report (SER), dated February 5, 2003 (NUREG-1769); (3) the licensee's Updated Final Safety Analysis Report; and (4) the Commission's Final Environmental Impact Statement, NUREG-1437, Supplement 10, dated January 22, 2003. These documents are available at the NRC's Public Document Room, One White Flint North, Public File Area 01 F21, 11555 Rockville Pike (first floor), Rockville, Maryland. Publicly available records will be available electronically from the Agency wide Documents Access and Management System (ADAMS). Public Electronic Reading Room on the internet at the NRC Web site <http://www.nrc.gov/reading-rm/adams.html>. Persons who do not have access to ADAMS or who encounter problems in accessing the documents located in ADAMS, should contact the NRC PDR Reference Staff by telephone at 1-800-397-4209, 301-415-4737 or by e-mail to [pdr@nrc.gov](mailto:pdr@nrc.gov).

Copies of Renewed Facility Operating License Nos. DPR-44 and DPR-56 may be obtained by writing to the U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, Attention: Director, Division of Regulatory Improvement Programs. Copies of the Safety Evaluation Report, NUREG-1769, and the Final Environmental Impact Statement, NUREG-1437, Supplement 10) may be purchased from the National Technical Information Service, Springfield, Virginia 22161-0002 (<http://www.ntis.gov>), 1-800-553-6847, or the Superintendent of Documents, U.S. Government Printing Office, P.O. Box 371954, Pittsburgh, PA 15250-7954 ([http://www.access.gpo.gov/su\\_docs](http://www.access.gpo.gov/su_docs)), 202-512-1800. All orders should clearly identify the NRC publication number and the requestor's Government Printing Office deposit account number or VISA or MasterCard number and expiration date.

Dated at Rockville, Maryland, this 7th day of May 2003.

For the Nuclear Regulatory Commission.

**Pao-Tsin Kuo,**

*Program Director, License Renewal and Environmental Impacts, Division of Regulatory Improvement Programs, Office of Nuclear Reactor Regulation.*

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## NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50-254 and 50-265]

### Exelon Generation Company, LLC; Notice of Consideration of Issuance of Amendment to Facility Operating License, Proposed No Significant Hazards Consideration Determination, and Opportunity for a Hearing

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License Nos. DPR-29 and DPR-30, issued to Exelon Generation Company, LLC (the licensee), for operation of the Quad Cities Nuclear Power Station (QCNPS), Units 1 and 2, located in Rock Island County, Illinois.

The proposed amendments would modify Technical Specification Surveillance Requirements to provide an alternative means of testing the Unit 1 main steam electromagnetic relief valves (ERVs), including those that provide the automatic depressurization and the low set relief functions, and provide an alternative means for testing the Units 1 and 2 dual function Target Rock safety/relief valves (S/RVs).

In its application for the exigent amendment, the licensee stated that on Unit 1, the 3A S/RV and 3C and 3D ERVs are currently leaking as evidenced by elevated tailpipe temperatures. The high tailpipe temperatures are indicative of steam leakage past the pilot valves or main valve seats. Leakage from ERVs and S/RVs is discharged to a point below the minimum water level in the suppression pool. Thus, the steam leakage can result in increasing suppression pool temperature. In addition, leakage past the pilot valves of S/RVs could cause an inadvertent opening of the main valve. Experience in the industry and at QCNPS indicates that manual actuation of main steam relief valves during plant operation can lead to increased seat leakage. As a result, the licensee plans as part of a maintenance outage previously scheduled for May 20, 2003, to replace the 3A S/RV. In addition, the 3C and 3D ERVs may also be replaced during the maintenance outage, pending results of additional testing to be performed at the start of the outage. This is being done based on the potential for steam leakage past the ERVs and S/RVs to result in increased suppression pool temperature. In addition, the alternative testing proposed for the 3A S/RV will reduce the potential for pilot valve leakage which can cause an inadvertent opening of the S/RV and impair the ability to re-close the valve. The need for this license

amendment was identified shortly following an inadvertent opening of a relief valve on Unit 2 that occurred April 16, 2003, and the S/RV and ERV work was added to the scope of the planned maintenance outage on April 23, 2003. The licensee states that it has used its best efforts to make a timely application for the amendment. To support plant startup following the outage, efforts to minimize the potential for increased suppression pool temperature caused by leaking relief valves, and the desire to minimize an inadvertent opening of an S/RV, the licensee requested NRC approval of the proposed changes by May 29, 2003. This need date precludes use of the normal 30-day notice period. Accordingly, as described above, the basis for an exigent amendment request exists and the current situation could not have been avoided.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

Pursuant to 10 CFR 50.91(a)(6) for amendments to be granted under exigent circumstances, the NRC staff must determine that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. Does the proposed change involve a significant increase in the probability or consequences of an accident previously evaluated?

*Response:* No.

The proposed changes modify Technical Specification (TS) Surveillance Requirement (SR) 3.4.3.2, SR 3.5.1.10, and SR 3.6.1.6.1 to provide an alternative means for testing the main steam line relief valves, automatic depressurization system valves, and low set relief valves. Accidents are initiated by the malfunction of plant equipment, or the catastrophic failure of plant structures, systems, or components. The performance of relief valve testing is not a precursor to any accident previously evaluated and does not change the manner in which the valves are operated. The proposed testing requirements will not contribute to the failure of the relief