

Therefore, the proposed changes do not involve a significant reduction in a margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

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NRC Section Chief: Robert A. Gramm.

Exelon Generation Company, LLC, Docket Nos. 50-237 and 50-249, Dresden Nuclear Power Station, Units 2 and 3, Grundy County, Illinois

Docket Nos. 50-254 and 50-265, Quad Cities Nuclear Power Station, Units 1 and 2, Rock Island County, Illinois

Date of application for amendment request: February 14, 2003.

Description of amendment request: The proposed amendments would relax the Technical Specifications (TSs) surveillance requirement (SR) for reactor instrumentation line excess flow check valves (EFCVs). Currently, TSs require testing of each reactor instrumentation line EFCV on a 24-month frequency. The proposed TS SR would require that a representative sample of reactor instrumentation line EFCVs be tested every 24 months, such that each EFCV will be tested nominally once every 10 years.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The proposed TS change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

The current Technical Specification (TS) Surveillance Requirement (SR) frequency requires each reactor instrumentation line excess flow check valve (EFCV) to be tested every 24 months. The EFCVs at Dresden Nuclear Power Station (DNPS) and Quad Cities Nuclear Power Station (QCNPS) are designed to remain open during normal operation, but will close automatically in the event of an instrument line break downstream of the valve. The proposed change allows a reduced number of reactor instrumentation line EFCVs to be tested every 24 months. Industry operating experience demonstrates a high level of reliability for these EFCVs. A failure of an EFCV to isolate cannot initiate previously evaluated accidents (*i.e.*, a break in a reactor coolant pressure boundary (RCPB)

instrument line outside containment).

Therefore, there is no increase in the probability of an accident as a result of this proposed change.

The postulated break of an instrument line connected to the RCPB is discussed and evaluated in the Updated Final Safety Analysis Reports (UFSARs) for DNPS and QCNPS. The integrity and functional performance of the secondary containment and standby gas treatment system are not impaired by this event, and the calculated potential offsite exposures are below the guidelines of 10 CFR 100, "Reactor Site Criteria." The NRC approved General Electric Nuclear Energy Licensing Topical Report, NEDO-32977-A, "Excess Flow Check Valve Testing Relaxation," discusses through operating experience that there is a high degree of reliability with the EFCVs and that there are little radiological consequences resulting from an EFCV failure. The radiological consequences for an instrument line break do not credit the EFCVs for isolating the break. Therefore, the consequences of an instrument line break are not impacted by the proposed level of testing. Based on the above, the proposed TS change does not involve a significant increase in the consequences of an accident previously evaluated.

In summary, the proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. The proposed TS change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

The proposed change allows a reduced number of reactor instrumentation line EFCVs to be tested every 24 months. No other changes in requirements are being proposed. Industry operating experience as documented in NEDO-32977-A, provides supporting evidence that the reduced testing will not affect the high reliability of these valves. The potential failure of an EFCV to isolate as a result of the proposed reduction in testing is bounded by the evaluation of an instrument line break described in the UFSARs for DNPS and QCNPS. The proposed changes do not physically alter the plant and will not alter the operation of structures, systems, and components as described in the UFSARs. Therefore, a new or different kind of accident from any accident previously evaluated will not be created.

3. The proposed TS change does not involve a significant reduction in a margin of safety.

The consequences of an unisolable rupture of a RCPB instrument line outside containment has been previously evaluated in the UFSARs for DNPS and QCNPS. That evaluation assumed a continuous discharge of reactor coolant for the duration of the detection and cooldown sequence (*i.e.*, no credit was assumed for isolating the break by the associated EFCV in the ruptured instrument line). Since a continuous discharge was assumed in this evaluation, any potential failure of the associated EFCV to isolate postulated by the reduced testing frequency is bounded. Therefore, the proposed change does not involve a significant reduction in a margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

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NRC Section Chief: Anthony J. Mendiola.

Exelon Generation Company, LLC, Docket Nos. 50-373 and 50-374, LaSalle County Station, Units 1 and 2, LaSalle County, Illinois

Date of amendment request: March 31, 2003.

Description of amendment request: The proposed amendments would revise Appendix A, Technical Specifications (TS), of Facility Operating License Nos. NPF-11 and NPF-18. Specifically, the proposed change will increase the upper limit associated with TS Table 3.3.5.1-1, "Emergency Core Cooling System Instrumentation," Function 3.e, "HPCS System Flow Rate—Low (Bypass)," Allowable Value from less than or equal to (\leq) 1704 gallons per minute (gpm) to \leq 2194 gpm. The proposed change increases the Allowable Value band to account for instrumentation deadband, as-left setting tolerances and setpoint drift, and resolves historical difficulties during calibration. The current Allowable Value was initially provided in the LaSalle County Station TS during conversion to Improved Technical Specifications (ITS) format. This value was based on vendor supplied data and believed at the time to adequately account for these parameters. The upper Allowable Value limit is being increased based on historical performance data for the High Pressure Core Spray (HPCS) system flow switches. The increase in the allowed bypass flow rate does not affect the capability of the HPCS system in performing its intended safety function.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The proposed change does not involve a significant increase in probability or consequences of an accident previously evaluated.

The proposed change to LaSalle County Station Technical Specifications (TS) Table 3.3.5.1-1, "Emergency Core Cooling System