

consideration, which is presented below:

1. Involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed changes do not alter the way any structure, system, or component functions and would not alter the way [in] which the plant is operated. The proposed changes do not involve any physical plant modifications. The proposed changes incorporate existing plant operational restrictions into the facility Technical Specifications and provide for the removal of information which is not applicable to plant operation.

The proposed allowed outage times (*i.e.* the required action times for Specification 3.4.1.5) are reasonable and consistent with the existing technical specification outage times and consistent with industry guidelines, thereby ensuring affected components are restored in a timely manner. The proposed changes to surveillance requirements are also consistent with existing surveillance frequencies and focus the Technical Specifications on verifying normal plant configurations are maintained. The design basis accidents, including the uncontrolled rod withdrawal from subcritical and boron dilution events, will remain the same postulated events described in the Millstone Unit No. 3 Final Safety Analysis Report (FSAR), and the consequences of these events will not be affected.

Therefore, the proposed changes will not increase the probability or consequences of an accident previously evaluated.

2. Create the possibility of a new or different kind of accident from any accident previously evaluated.

The proposed changes do not alter the plant configuration (no new or different type of equipment will be installed) or require any new or unusual operator actions. The proposed changes do not alter the way any structure, system, or component functions and do not alter the manner in which the plant is operated. The proposed changes do not introduce any new failure modes. Therefore, the proposed changes do not create the possibility of a new or different kind of accident from any accident previously evaluated.

3. Involve a significant reduction in a margin of safety.

The proposed changes will not reduce the margin of safety since they have no impact on any accident analysis assumption. The proposed changes do not decrease the scope of equipment currently required to be OPERABLE or subject to surveillance testing, nor do the proposed changes affect any instrument setpoints or equipment safety functions. The effectiveness of Technical Specifications will be maintained since the changes will not alter the operation of any component or system, nor will the proposed changes affect any safety limits or safety system settings which are credited in a facility accident analysis. Therefore, there is no reduction in a margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three

standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

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NRC Section Chief: James W. Clifford.

Duke Energy Corporation, Docket Nos. 50-269, 50-270, and 50-287, Oconee Nuclear Station, Units 1, 2, and 3, Oconee County, South Carolina

Date of amendment request: December 4, 2002.

Description of amendment request: The amendments revise Technical Specification 3.7.6 by changing the minimum combined inventory for Emergency Feedwater from 72,000 gallons to 155,000 gallons and eliminating the condensate storage tank as a source of this inventory.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

Pursuant to 10 CFR 50.91, Duke Power Company (Duke) has made the determination that this amendment request involves a No Significant Hazards Consideration by applying the standards established by the NRC regulations in 10 CFR 50.92. This ensures that operation of the facility in accordance with the proposed amendment would not:

(1) Involve a significant increase in the probability or consequences of an accident previously evaluated:

No. This revision to Technical Specification (TS) 3.7.6 changes the inventory requirements for the Upper Surge Tank (UST) and hotwell. These components provide a suction source to the Emergency Feedwater System (EFW). This increase in inventory from 72,000 gallons to 155,000 gallons increases the required available inventory. This increase in inventory does not affect the probability or consequences of any previously evaluated accident.

(2) Create the possibility of a new or different kind of accident from any kind of accident previously evaluated:

No. This revision to the combined UST and hotwell inventory increases the required amount of water available to the EFW system. No new or different kind of accident is created by this change as only the required inventory is revised.

(3) Involve a significant reduction in a margin of safety:

No. The increase in required UST and hotwell inventory does not reduce the margin of safety. The increase provides the required inventory to ensure that the EFW can provide a Reactor Coolant System cooldown at a rate of 50° F/hour to decay heat removal entry conditions following a reactor trip.

Duke has concluded, based on the above, that there is no significant hazards considerations involved in this amendment request.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

Attorney for licensee: Anne W. Cottingham, Winston and Strawn, 1200 17th Street, NW., Washington, DC 20005.

NRC Section Chief: John A. Nakoski.

Entergy Nuclear Operations, Inc., Docket No. 50-293, Pilgrim Nuclear Power Station, Plymouth County, Massachusetts

Date of amendment request: May 1, 2002, as supplemented December 4, 2002.

Description of amendment request: The proposed amendment would extend the applicability of the current Pilgrim Nuclear Power Station (Pilgrim) reactor pressure vessel pressure-temperature (P-T) curves through the end of Operating Cycle (OC) 16. The current P-T curves were approved for use in License Amendment 190, dated April 13, 2001, and are limited to use through the end of OC 14. The proposed change would delete the 20 and 32 Effective Full Power Year (EFPY) curves and replace the wording of the title blocks to allow use through the end of OC 16. The proposed amendment would change Pilgrim Technical Specification Figures 3.6.1, 3.6.2 and 3.6.3.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration. The NRC staff has reviewed the licensee's analysis against the standards of 10 CFR 50.92(c). The NRC staff's review is presented below:

1. Does the proposed change involve a significant increase in the probability or consequences of an accident previously evaluated?

The proposed change involves a request to extend the use of the current reactor pressure vessel P-T curves for two additional OCs. The P-T curves were generated in accordance with the fracture toughness requirements of 10 CFR Part 50, Appendix G, and American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code), Section XI, Appendix G and Regulatory Guide 1.99, Revision 2, Radiation Embrittlement of Reactor Vessel Materials, and were established in