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AmerGen Energy Company, LLC, et al., Docket No. 50-219, Oyster Creek Nuclear Generating Station, Ocean County, New Jersey

Date of amendment request: April 21, 2003.

Description of amendment request: The licensee proposed to revise Sections 3.7 and 4.7, "Auxiliary Electrical Power," of the Technical Specifications (TSs) to make them generally consistent with Nuclear Regulatory Commission (NRC) guidance set forth in NUREG-1433, "Standard Technical Specifications, General Electric Plants, BWR [Boiling Water Reactor]/4," Revision 2, and with the NRC-approved industry guidance identified as Technical Specification Task Force (TSTF) traveler TSTF-360, Revision 1. The amendment would also add a new Section 6.8.5, "Station Battery Monitoring and Maintenance Program." The resulting Sections 3.7, 4.7, and 6.8.5 will be explicitly applicable to station batteries B and C, both safety-related subsystems, and their associated battery chargers. The proposed amendment would revise requirements concerning surveillance, monitoring, and maintenance of the subject batteries and chargers.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration. The NRC staff has reviewed the licensee's analysis against the three standards of 10 CFR 50.92(c) and performed its own. The NRC staff's analysis is presented below:

The first standard requires that operation of the unit in accordance with the proposed amendment will not involve a significant increase in the probability or consequences of an accident previously evaluated. The proposed changes, if approved by the NRC, will be made in a manner such that conservatism is maintained through

compliance with applicable NRC regulations and guidance. No hardware design change is involved with the proposed amendment, thus there will be no adverse effect on the functional performance of any plant structure, system, or component (SSC). Consequently, all SSCs will continue to perform their design functions with no decrease in their capabilities to mitigate the consequences of postulated accidents. Station battery surveillance, monitoring, and maintenance were not previously factored into the probability of accidents, nor were they factored into scenarios of previously analyzed accidents. Consequently, the proposed revision to Sections 3.7, 4.7, and 6.8.5 of the TSs will lead to no increase in the consequences of an accident previously evaluated, and no increase of the probability of an accident previously evaluated.

The second standard requires that operation of the unit in accordance with the proposed amendment will not create the possibility of a new or different kind of accident from any accident previously evaluated. The proposed amendment is not the result of a hardware design change, nor does it lead to the need for a hardware design change. There is no change in the methods the unit is operated. As a result, all SSCs will continue to perform as previously analyzed by the licensee, and previously evaluated and accepted by the NRC staff. Therefore, the proposed amendment will not create the possibility of a new or different kind of accident from any previously evaluated.

The third standard requires that operation of the unit in accordance with the proposed amendment will not involve a significant reduction in a margin of safety. Since the licensee did not propose to exceed or alter a design basis or safety limit, the proposed amendment will not affect in any way the performance characteristics and intended functions of any SSC. Therefore, the proposed amendment does not involve a significant reduction in a margin of safety.

Based on the NRC staff's analysis, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

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Arizona Public Service Company, et al., Docket Nos. STN 50-528, STN 50-529, and STN 50-530, Palo Verde Nuclear Generating Station (PVNGS), Units 1, 2, and 3, Maricopa County, Arizona

Date of amendments request: April 15, 2003.

Description of amendments request: The amendments would revise Sections 2.2, "SL [Safety Limits] Violations," for reporting such violations to positions in the plant organization; 5.2.1, "Onsite and Offsite Organization," for the position responsible for overall safe plant operation; and 5.5.1, "Offsite Dose Calculation Manual (ODCM)," for the position that approves changes to the ODCM, of the Technical Specifications (TSs). The revisions would account for the elimination of the positions of Vice President, Nuclear Production, and Director, Site Chemistry, and the assignment of the responsibilities of these positions in the above TS sections to other positions in the plant organization. Also, there would be the format change of adding the title of Section 2.2 near the top of TS page 2.0-2.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

These changes involve minor changes in the organization of PVNGS. It is expected that the organizational changes will have a positive effect on the conduct of plant operations and safety-related work. Functions which are necessary to operate the facility safely and in accordance with the operating licenses, remain in the re-aligned organization and will not affect the safe operation of the plant and continue to ensure proper control of administrative activities. The Quality Assurance (QA) organization reporting structure has not been affected by these changes allowing the QA organization to maintain the required authority and organizational freedom to identify quality problems; to initiate, recommend, or provide solutions; and to verify implementation of solutions. The proposed changes will not affect the operation of structures, systems, [or] components, and will not reduce programmatic controls such that plant safety would be affected. (The changes in the plant organization are also not initiators of an accident.) Therefore, the proposed changes do not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. The proposed change does not create the possibility of a new or different kind of accident from any accident previously evaluated.