

different kind of accident from any accident previously evaluated.

3. Involve a significant reduction in a margin of safety.

The proposed changes have no impact on plant equipment operation. The proposed changes do not revise any setpoints assumed in the analyses and do not affect the acceptance criteria for SBLOCA analyses. Therefore, the proposed changes will not result in a reduction in a margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

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NRC Section Chief: James W. Clifford.

Duke Energy Corporation, Docket Nos. 50-269, 50-270, and 50-287, Oconee Nuclear Station, Units 1, 2, and 3, Oconee County, South Carolina

Date of amendment request: April 10, 2003.

Description of amendment request: The proposed amendments would revise the Technical Specifications (TS) for the low temperature overpressure protection system. Currently, TS Surveillance Requirement (SR) 3.4.12.5 requires performance of a channel functional test for the power-operated relief valve within 12 hours of decreasing reactor coolant system (RCS) temperature to $\leq 325^\circ\text{F}$ and every 31 days thereafter. The proposed amendments would revise TS SR 3.4.12.5 to allow the first performance of this surveillance to be within 31 days prior to decreasing RCS temperature to $\leq 325^\circ\text{F}$. The proposed amendments also would revise the frequency of the channel calibration in TS SR 3.4.12.7 from 18 months to 6 months.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

Pursuant to 10 CFR 50.91, Duke Power Company (Duke) has made the determination that this amendment request involves a No Significant Hazards Consideration by applying the standards established by the NRC regulations in 10 CFR 50.92. This ensures that operation of the facility in accordance with the proposed amendment would not:

(1) Involve a significant increase in the probability or consequences of an accident previously evaluated;

This is a revision to the Technical Specification (TS) surveillance requirement (SR) for performing the channel functional test (CFT) for the pressurizer [power] [...] operated relief valve (PORV). As such, changing the requirement to perform the first CFT before entering the Low Temperature Overpressure Protection (LTOP) region, rather than after LTOP is required, eliminates removing the PORV from service, in the mode of applicability for the performance of the CFT. This change will decrease the probability of a low temperature overpressurization of the reactor vessel, thereby increasing safety and reducing risk, by maintain(ing) both trains (active and passive) of the LTOP System operable. The change to the frequency for performance of SR 3.4.12.7 is being done to ensure the calibration is performed in a time frame supported by current analysis. The method of test is not changed, only the frequency. This reduction in frequency will not significantly increase the probability or consequences of any accident previously evaluated.

(2) Create the possibility of a new or different kind of accident from any kind of accident previously evaluated:

This revision will not impact the LTOP evaluation analysis. The timeframe to perform the CFT for the PORV will not change the operation of the PORV or its function during accident conditions. No new or different accidents result from performing the CFT prior to entering LTOP conditions. The revision to SR 3.4.12.7 only changes the frequency of the testing. The method of test is not changed. This change has no effect on the possibility of a new or different kind of accident.

(3) Involve a significant reduction in a margin of safety:

The proposed revision will perform the CFT within 31 days prior to entering LTOP conditions, rather than performing the test once LTOP conditions are entered. This allows the CFT, which causes the PORV to be inoperable for a short period of time, to be performed prior to reaching the plant conditions where the PORV is relied upon for LTOP. Performing the CFT within 31 days prior to decreasing RCS temperature to $< 325^\circ\text{F}$, rather than after entering these conditions, will not change the margin of safety. Oconee calculations show that a recalibration interval of 6 months for the Reactor Coolant System (RCS) low range pressure instrumentation results in a single-sided 95/95 probability confidence limit of 9.4 psig. This result is bounded by the instrument uncertainty assumed in the LTOP evaluation analysis. The frequency change for SR 3.4.12.7 from 18 months to 6 months does not affect the method of test performance. It only decreases the allowed time between performances to reflect current Oconee analysis. This will not significantly reduce the margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

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NRC Section Chief: John A. Nakoski.

Entergy Nuclear Operations, Inc., Docket No. 50-293, Pilgrim Nuclear Power Station, Plymouth County, Massachusetts

Date of amendment request: July 5, 2002, as supplemented August 13, 2002.

Description of amendment request: The proposed amendment would relocate portions of Technical Specification (TS) 3/4.6.B, "Primary System Boundary—Coolant Chemistry," from the TSs to the Updated Final Safety Analysis Report (UFSAR). The portions of the TS that would be relocated to the UFSAR are the reactor coolant chemistry requirements for conductivity and chloride concentration. Specifically, TSs 3/4.6.B.2, 3/4.6.B.3, and 3.6.B.4 would be relocated to the UFSAR.

Basis for proposed no significant hazards consideration determination: As required by Title 10 of the Code of Federal Regulations (10 CFR) 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. Does the proposed change involve a significant increase in the probability or consequences of an accident previously evaluated.

Response: No. The proposed change is administrative in nature and does not involve the modification of any plant equipment or affect basic plant operation. Conductivity and chloride limits are not assumed to be an initiator of any analyzed event, nor are these limits assumed in the mitigation of consequences of accidents.

Therefore, the proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. Does the proposed change create the possibility of a new or different kind of accident from any accident previously evaluated?

Response: No. The proposed change does not involve any physical alteration of plant equipment and does not change the method by which any safety-related system performs its function. As such, no new or different types of equipment will be installed, and the basic operation of installed equipment is unchanged. The methods governing plant operation and testing remain consistent with current safety analysis assumptions. Therefore, the proposed change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

3. Does the proposed change involve a significant reduction in a margin of safety?

Response: No. The proposed change represents the relocation of current Technical