

I&M proposes changing the low pressurizer pressure SI trip setpoint, the low pressurizer pressure SI allowable value, the P-11 setpoint, and the format of the associated pages. Neither the change to the low pressurizer pressure SI trip setpoint value and the SI allowable value nor the change to the P-11 setpoint value involve changing the design function of any component, and a change in any of the values cannot initiate an accident. The format changes are intended to improve readability and appearance, and do not alter any requirements. Thus, no new accident initiators are introduced, and the possibility of a new or different kind of accident is not created.

3. Does the proposed change involve a significant reduction in a margin of safety?
Response: No.

I&M proposes changing the low pressurizer pressure SI trip setpoint, the low pressurizer pressure SI allowable value, the P-11 setpoint, and the format of the associated pages. The low pressurizer pressure instrument is credited for activating the engineered safety features in the event of a LOCA, a main steam system depressurization event, or a feedwater line break. The low pressurizer pressure SI trip setpoint value and the low pressurizer pressure SI allowable value have been selected to insure that the engineered safety features will be activated as assumed in the safety analysis. Present margins continue to be maintained because the applicable accident analyses criteria continue to be met. No margins of safety are associated with the P-11 setpoint value. The format changes are intended to improve readability and appearance, and do not alter any requirements. Thus, there is no significant reduction in a margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment requests involve no significant hazards consideration.

Attorney for licensee: David W. Jenkins, Esq., 500 Circle Drive, Buchanan, MI 49107.

NRC Section Chief: L. Raghavan.

Maine Yankee Atomic Power Company, Docket No. 50-309, Maine Yankee Atomic Power Station, Lincoln County, Maine

Date of amendment request: April 24, 2003.

Description of amendment request: Eliminate the requirement for continuous Control Room manning when fuel is stored in the fuel storage pool.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. Does the proposed amendment involve a significant increase in the probability or consequences of an accident previously evaluated? Involve a significant increase in the probability or consequence of an accident previously evaluated.

Response: No.

The Defueled Safety Analysis (DSAR) identifies five categories of events: spent fuel criticality accidents, a fuel handling accident, a spent fuel cask drop, spent fuel pool accidents, and a low level waste release incident. There are no active controls in the control room that are required to respond to these events. Actions to mitigate the consequences of these events are taken outside the control room. Emergency response is not adversely affected by this proposed change because the control room is still available to the emergency response team and communications capability and timeliness will not be affected. Therefore, the proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. Does the proposed amendment create the possibility of a new or different kind of accident from any accident previously evaluated?

Response: No.

The configuration, operations and accident response of the systems, structures or components that support safe storage of the spent fuel are unchanged by the proposed change to the technical specifications. Current site surveillance requirements ensure frequent and adequate monitoring of system and component functionality. Systems in the Spent Fuel Pool Island will continue to be operated in accordance with current design requirements and no new components or system interactions have been identified. No new accident scenarios, failure mechanisms or limiting single failures are introduced as a result of the proposed change. The proposed technical specification change does not have an adverse affect on any system related to safe storage of spent fuel. Therefore, the proposed technical specifications change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

3. Does the proposed amendment involve a significant reduction in a margin of safety?

Response: No.

All design basis accident acceptance criteria will continue to be met. The margin of safety relative to the cooling of the spent fuel is unaffected by the proposed changes as the spent fuel pool parameters will continue to be monitored at the same frequency as assumed in the accident analysis. The ability of the shift crew to respond to abnormal or accident conditions is unaffected by the proposed change since all controls are located in or near the fuel building and any necessary communications will be handled by the on-shift staff and/or DERO. Therefore, it is concluded that the proposed TS change does not involve a significant reduction in the margin of safety.

Based on the above, Maine Yankee concludes that the proposed amendment presents no significant hazards consideration

under the standards set forth in 10 CFR 50.92(c), and, accordingly, a finding of "no significant hazards consideration" is justified.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the requested amendment involves no significant hazards consideration.

Attorney for licensee: Joe Fay, Esquire, Maine Yankee Atomic Power Company, 321 Old Ferry Road, Wiscasset, Maine 04578.

NRC Section Chief: Claudia M. Craig.

Nebraska Public Power District, Docket No. 50-298, Cooper Nuclear Station, Nemaha County, Nebraska

Date of amendment request: October 8, 2002.

Description of amendment request: The license amendment request proposes to change the title of (a) Shift Supervisor to Shift Manager, (b) "Plant Manager" to "plant manager," (c) "Vice President—Nuclear" to "corporate officer with direct responsibility for the plant," (d) "Radiological Manager" to "radiological manager," (e) "Operations Supervisor" to "operations supervisor" and (f) "Shift Radiological Protection/Chemistry Technician" to "radiation protection technician." This proposal includes an Updated Safety Analysis Report (USAR) reference correction resulting from the USAR Rebaseline Project and a correction to the title "Shift Technical Advisor" to "Shift Technical Engineer" in Technical Specification (TS) Section 5.3.1 so as to be consistent with the title used in TS Section 5.2.2.f. These changes do not eliminate any of the qualifications, responsibilities, or requirements for these positions.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. Do the proposed changes involve a significant increase in the probability or consequences of an accident previously evaluated?

The title of Shift Manager better conveys the appropriate level of responsibility and authority required of the position. The use of generic personnel titles and correction of the USAR reference are strictly administrative. The qualifications, training, duties and experience required of the individuals remain unchanged. The USAR section to be referenced is physically the same section that was referenced before the USAR renumbering. The requested changes do not