

involve any change to the design basis of the plant or any structure, system, or component. Therefore, these changes do not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. Do the proposed changes create the possibility of a new or different kind of accident from any accident previously evaluated?

There will be no physical alterations to the plant configuration. No changes in operating mode or limits are proposed. The qualifications, training, duties and experience required of the individuals remain unchanged. The USAR section to be referenced is physically the same section that was referenced before the USAR renumbering. Therefore, these proposed changes do not create the possibility of a new or different kind of accident from any previously evaluated.

3. Do the proposed changes involve a significant reduction in the margin of safety?

The proposed change in titles and USAR reference are strictly administrative. The qualifications, training, duties and experience required of the individuals remain unchanged. The USAR section to be referenced is physically the same section that was referenced before the USAR renumbering. The proposed changes do not change any license condition or Technical Specifications safety limit or limiting condition for operation. The changes do not involve modification of the design or operation of any plant system involved with controlling the release of radioactivity to the environment. Therefore, these changes do not involve a significant reduction in a margin of safety.

Based on the above, Nebraska Public Power District concludes that the proposed amendment presents no significant hazards consideration under the standards set forth in 10 CFR 50.92(c), and accordingly, a finding of "no significant hazards consideration" is justified.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

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Nuclear Management Company, LLC, Docket No. 50-331, Duane Arnold Energy Center, Linn County, Iowa

Date of amendment request: May 2, 2003.

Description of amendment request: The proposed amendment would change the Technical Specifications (TSs) by replacing the existing Reactor Coolant System (RCS) pressure and temperature (P/T) limit curves for in-

service leakage and hydrostatic testing, non-nuclear heatup and cooldown, and criticality (Figure 3.4.9-1, "Pressure Versus Minimum Temperature Valid to Thirty-two Full Power Years, per Appendix G of 10 CFR 50") with new, updated P/T limits curves. The replacement curves were generated using an NRC-approved methodology (General Electric Report NEDC-32983P) for determining the neutron fluence on the Reactor Pressure Vessel (RPV) and extends the RPV beltline region to encompass a new limiting component, the recirculation inlet nozzle. The change to Figure 3.4.9-1 would also delete the existing notation that states: "(Interim Approval Until September 1, 2003)."

The licensee's application for amendment dated May 2, 2003, supersedes and withdraws a previous application dated February 28, 2003, for which the NRC has published a notice of consideration of issuance of amendment, proposed no significant hazards consideration determination, and opportunity for hearing in the **Federal Register** (68 FR 12954, dated March 18, 2003).

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

(1) The proposed amendment will not involve a significant increase in the probability or consequences of an accident previously evaluated.

The P/T limits are not derived from Design Basis Accident (DBA) analyses. They are prescribed by the (American Society of Mechanical Engineers Boiler and Pressure Vessel Code) ASME Code and 10 CFR 50 Appendix G and H and associated guidance documents, such as Regulatory Guide 1.99, Rev. 2, as restrictions on normal operation to avoid encountering pressure, temperature, and temperature rate of change conditions that might cause undetected flaws to propagate and cause non-ductile failure of the reactor coolant pressure boundary. Thus, they ensure that an accident precursor is not likely. Hence, they are included in the TS as satisfying Criterion 2 of 10 CFR 50.36(c)(2)(ii). The revision of the numerical value of these limits, *i.e.*, new curves, using an NRC-approved methodology, does not change the existing regulatory requirements, upon which the curves are based. Thus, this revision will not increase the probability of any accident previously evaluated.

The proposed change does not alter the design assumptions, conditions, or configuration of the facility or the manner in which the facility is operated or maintained. The proposed changes will not affect any other System, Structure or Component (SSC) designed for the mitigation of previously analyzed events. The proposed change does

not affect the source term, containment isolation, or radiological release assumptions used in evaluating the radiological consequences of any accident previously evaluated. Thus, the proposed revision of the existing numerical values with the updated figure for the RCS P/T limits, which are based upon an NRC-approved methodology for calculating the neutron fluence on the RPV and new limiting component, will not increase the consequences of any previously evaluated accident.

(2) The proposed amendment will not create the possibility of a new or different kind of accident from any accident previously evaluated.

The proposed changes do not involve a physical alteration of the plant (*i.e.*, no new or different type of equipment will be installed) or a change in the processes governing normal plant operation. The proposed changes are consistent with the safety analysis assumptions and current plant operating practice. (Nuclear Management Company, LLC) NMC is only requesting to revise the existing numerical values and update the TS figure for the RCS P/T limits based upon an NRC-approved methodology for calculating the neutron fluence on the RPV, and to reflect a new limiting component. The curves continue to be based upon ASME Code Case N-640, which has been previously approved for use at the [Duane Arnold Energy Center] DAEC.

Therefore, the proposed changes do not create the possibility of a new or different kind of accident from any previously evaluated.

(3) The proposed amendment will not involve a significant reduction in a margin of safety.

The proposed changes do not alter the manner in which Safety Limits, Limiting Safety System Settings or Limiting Conditions for Operation are determined. The setpoints at which protective actions are initiated are not altered by the proposed changes. Sufficient equipment remains available to actuate upon demand for the purpose of mitigating an analyzed event. NMC is only requesting to revise the existing numerical values and update the TS figure for the RCS P/T limits based upon an NRC-approved methodology for calculating the neutron fluence, NEDC-32983P-A. The new curves also reflect the addition of a new limiting component, the recirculation inlet nozzle (N2). No other changes to the Limiting Conditions for Operation or any Surveillance Requirements of Technical Specification 3.4.9 are proposed.

10 CFR 50, Appendix G specifies fracture toughness requirements to provide adequate margins of safety during operation over the service lifetime. The values of adjusted reference temperature and upper shelf energy are expected to remain within the limits of Regulatory Guide 1.99, Revision 2 and Appendix G of 10 CFR 50 for at least 32 effective full power years (EFPY) of operation. The safety analysis supporting this change continues to satisfy the ASME Code, including ASME Code Case N-640, and 10 CFR 50, Appendices G and H requirements and associated guidance documents, such as Regulatory Guide 1.99, Rev. 2. Thus, the