

different kind of accident from any previously evaluated.

3. Does the proposed change involve a significant reduction in a margin of safety?

*Response:* No.

The generic study of the increase in the Type A test interval, NUREG-1493, concluded there is an imperceptible increase in the plant risk associated with extending the test interval out to twenty years. The evaluations done in support of this change confirm that (conclusion). Further, the extended test interval would have a minimal effect on this risk since Type B and C testing detects 97% of potential leakage paths. For the requested change in the GGNS ILRT/DWBT interval, it was determined that the risk contribution of leakage will increase 0.99%. This change is considered very small and does not represent a significant reduction in the margin of safety.

Therefore, the proposed change does not involve a significant reduction in a margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

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*NRC Section Chief:* Robert A. Gramm.

*Exelon Generation Company, LLC, Docket Nos. 50-373 and 50-374, LaSalle County Station, Units 1 and 2, LaSalle County, Illinois*

*Date of amendment request:* April 18, 2003.

*Description of amendment request:* The proposed amendments would revise Appendix A, Technical Specifications (TS), of Facility Operating License Nos. NPF-11 and NPF-18. Specifically, the proposed change will modify TS Table 3.3.6.1-1, "Primary Containment Isolation Instrumentation," to add the requirement to perform a Channel Check in accordance with Surveillance Requirement (SR) 3.3.6.1.1 to thirteen listed instrument functions. The proposed change is the result of the replacement of existing plant equipment with equipment that has the capability of permitting the performance of a Channel Check with the plant in MODE 1, 2, and 3. The proposed change is consistent with the wording specified in NUREG-1434, "Standard Technical Specifications General Electric Plants, BWR/6," Revision 2, dated June 2001.

*Basis for proposed no significant hazards consideration determination:*

As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The proposed change does not involve a significant increase in probability or consequences of an accident previously evaluated.

The proposed change to Technical Specifications (TS) Table 3.3.6.1-1, "Primary Containment Isolation Instrumentation" will incorporate into the LaSalle County Station (LSCS) TS, wording specified in NUREG-1434, "Standard Technical Specifications General Electric Plants, BWR/6," Revision 2, dated June 2001. The proposed change will modify TS Table 3.3.6.1-1 to add the requirement to perform a Channel Check in accordance with Surveillance Requirement (SR) 3.3.6.1.1 to thirteen listed instrument functions. The performance of TS surveillance testing is not a precursor to any accident previously evaluated. A Channel Check is a monitoring activity that does not represent an accident initiator. Thus, the proposed change does not have any effect on the probability of an accident previously evaluated.

The function of instrumentation listed on TS Table 3.3.6.1-1, in combination with other accident mitigation features, is to limit fission product release during and following postulated Design Basis Accidents (DBAs) to within limits. The surveillance testing specified in TS Table 3.3.6.1-1 will provide assurance that the instrumentation will perform as designed. Thus, the radiological consequences of any accident previously evaluated are not increased.

Therefore, the proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. The proposed change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

The proposed change does not affect the control parameters governing unit operation or the response of plant equipment to transient conditions. The failure modes of the new instrumentation do not give rise to a new or different kind of accident. The proposed change does not introduce any new modes of system operation or failure mechanisms.

Therefore, the proposed change does not create the possibility of a new or different kind of accident from any previously evaluated.

3. The proposed changes do not involve a significant reduction in a margin of safety.

The leak detection system at LaSalle County Station uses ambient or differential temperature increases to detect small primary coolant boundary leaks in the Main Steam Line Tunnel and in various rooms of the Reactor Core Isolation Cooling (RCIC) System and the Reactor Water Cleanup (RWCU) System. The existing thermocouple monitors did not have the capability to allow a Channel Check to be performed without undue risk of initiating an inadvertent system

isolation in MODE 1, 2 and 3. Thus, the LSCS TS took exception to the guidance contained in NUREG-1434 and did not specify on TS Table 3.3.6.1-1 that a SR 3.3.6.1.1 Channel Check be performed on the above listed thirteen instrument functions.

The new thermocouple monitors have continuously reading digital displays that permit the performance of a Channel Check with the Unit in MODE 1, 2 and 3 without risk of inadvertent system isolations. The new thermocouple digital displays have been installed on Unit 2 during the January/February 2003 refuel outage and are scheduled to be installed in Unit 1 during the upcoming January 2004 refuel outage. LSCS after the return to service of Unit 2 in March of 2003, verified that the thermocouple digital displays do permit a Channel Check to be successfully performed on the above listed thirteen instrument functions. Therefore, LSCS is requesting that TS Table 3.3.6.1-1 is modified to specify that a SR 3.3.6.1.1 Channel Check be performed in MODE 1, 2 and 3, consistent with the guidance contained in NUREG-1434, Rev. 2.

Therefore, the proposed changes do not involve a significant reduction in a margin of safety.

Based upon the above, Exelon Generation Company concludes that the proposed amendment presents no significant hazards consideration under the standards set forth in 10 CFR 50.92(c), and, accordingly, a finding of "no significant hazards consideration" is justified.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the requested amendments involve no significant hazards consideration.

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*NRC Section Chief:* Anthony J. Mendiola.

*Exelon Generation Company, LLC, Docket Nos. 50-373 and 50-374, LaSalle County Station, Units 1 and 2, LaSalle County, Illinois*

*Date of amendment request:* April 18, 2003.

*Description of amendment request:* The proposed amendments would revise Appendix A, Technical Specifications (TS), of Facility Operating License Nos. NPF-11 and NPF-18. Specifically, the proposed change will modify TS Surveillance Requirement (SR) 3.6.1.3.8 to identify that the specified testing requirement is applicable to reactor instrumentation lines. The proposed change is consistent with the SR wording specified in