

greater than 110 tons with the reactor building crane during power operation. The load drop analysis demonstrates that dropping a reactor cavity shield block within the designated safe load path from the heights assumed in the analysis will not affect the capability of safety-related equipment to perform its function. Therefore, it is concluded that the proposed change does not result in a significant reduction in the margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the requested amendments involve no significant hazards consideration.

Attorney for licensee: Mr. Edward J. Cullen, Deputy General Counsel, Exelon BSC—Legal, 2301 Market Street, Philadelphia, PA 19101.

NRC Section Chief: Anthony J. Mendiola.

Exelon Generation Company, LLC, Docket Nos. 50-373 and 50-374, LaSalle County Station, Units 1 and 2, LaSalle County, Illinois.

Date of amendment request: May 19, 2003.

Description of amendment request: The proposed amendments would revise Appendix A, Technical Specifications (TS), of Facility Operating License Nos. NPF-11 and NPF-18. Specifically, the proposed change will decrease the frequency associated with TS Surveillance Requirement (SR) 3.7.7.1 for Turbine Bypass Valve (BPV) testing from 7 to 31 days. The proposed change is consistent with the testing frequency contained in NUREG-1434, "Standard Technical Specifications General Electric Plants, BWR/6," Revision 2, dated June 2001, for BPV testing.

The 7-day frequency associated with SR 3.7.7.1 was established in the LaSalle County Station (LSCS) TS during conversion to Improved Technical Specifications (ITS) format due to the testing frequency contained in the LSCS custom TS and the difficulties experienced with other Electro-Hydraulic Control (EHC) system valves to consistently pass their surveillance tests. LSCS has recently re-evaluated the performance of these valves and has determined that the current performance of these valves supports decreasing the testing frequency of the BPVs from 7 to 31 days.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards

consideration, which is presented below:

1. The proposed change does not involve a significant increase in probability or consequences of an accident previously evaluated.

The proposed change will decrease the frequency associated with Surveillance Requirement (SR) 3.7.7.1 for turbine bypass valve (BPV) testing from 7 to 31 days. The proposed change is consistent with the testing frequency contained in NUREG-1434, "Standard Technical Specifications General Electric Plants, BWR/6," Revision 2, dated June 2001, for BPV testing. The performance of BPV surveillance testing is not a precursor to any accident previously evaluated.

Thus, the proposed change does not have any effect on the probability of an accident previously evaluated.

The Main Turbine Bypass System is required to be operable to limit peak pressure in the main steam lines and maintain reactor pressure within acceptable limits during events that cause rapid pressurization, such that the Safety Limit Minimum Critical Power Ratio (MCPR) is not exceeded. An operable Main Turbine Bypass System requires the BPVs to open in response to increasing main steam line pressure. The performance of BPVs surveillance testing provides assurance that the valves will operate as assumed in accidents previously evaluated. Thus, the radiological consequences of any accident previously evaluated are not increased.

Therefore, the proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. The proposed change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

The proposed change does not affect the control parameters governing unit operation and does not introduce any new equipment, modes of system operation or failure mechanisms. Therefore, the proposed change does not create the possibility of a new or different kind of accident from any previously evaluated.

3. The proposed changes do not involve a significant reduction in a margin of safety.

The proposed change will decrease the frequency associated with SR 3.7.7.1 for BPV testing from 7 to 31 days. The proposed change is consistent with the BPV testing frequency contained in NUREG-1433, Revision 2, and does not affect the design parameters or the setpoints associated with BPV operation. Therefore, the proposed changes do not involve a significant reduction in a margin of safety.

Based upon the above, Exelon Generation Company concludes that the proposed amendment presents no significant hazards consideration under the standards set forth in 10 CFR 50.92(c), and, accordingly, a finding of "no significant hazards consideration" is justified.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are

satisfied. Therefore, the NRC staff proposes to determine that the requested amendments involve no significant hazards consideration.

Attorney for licensee: Mr. Edward J. Cullen, Deputy General Counsel, Exelon BSC—Legal, 2301 Market Street, Philadelphia, PA 19101.

NRC Section Chief: Anthony J. Mendiola.

FirstEnergy Nuclear Operating Company, et al., Docket Nos. 50-334 and 50-412, Beaver Valley Power Station, Unit Nos. 1 and 2, Beaver County, Pennsylvania

Date of amendment request: March 26, 2003.

Description of amendment request: The proposed amendments would modify Technical Specifications (TSs) 4.0.1 and 4.0.3 to be consistent with the Improved Standard Technical Specifications. The proposed amendments would also modify the TS requirements for missed surveillances in TS 4.0.3 to be consistent with the Nuclear Regulatory Commission (NRC)-approved Technical Specification Task Force (TSTF), Standard Technical Specification Change TSTF-358, Revision 6.

The NRC staff issued a notice of opportunity for comment in the **Federal Register** on June 14, 2001 (66 FR 32400), on possible amendments concerning missed surveillances, including a model safety evaluation and model no significant hazards consideration (NSHC) determination, using the consolidated line item improvement process. The NRC staff subsequently issued a notice of availability of the models for referencing in license amendment applications in the **Federal Register** on September 28, 2001 (66 FR 49714). The licensee affirmed the applicability of that portion of the following NSHC determination, related to the adoption of the TSTF-358, Revision 6, changes to the TSs in its application dated March 26, 2003.

Basis for proposed no significant hazards consideration determination:

Item 1: Modification of TSs 4.0.1 and 4.0.3 to be consistent with the Improved Standard Technical Specifications.

As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. Does the proposed change involve a significant increase in the probability or consequences of an accident previously evaluated?

Response: No.

The proposed changes involve rewording of the existing Technical Specifications to be