

Omaha Public Power District, Docket No. 50-285, Fort Calhoun Station, Unit No. 1, Washington County, Nebraska
Date of amendment request: October 8, 2002.

Description of amendment request: Omaha Public Power District (OPPD) has proposed the following changes to the Technical Specifications (TSs): (1) Use a pressure temperature limits report (PTLR), (2) change the minimum boltup temperature, (3) modify the TSs to reflect the revised low temperature overpressure protection (LTOP) methodology and analysis that is submitted for review and approval, (4) perform LTOP analyses "in-house," (5) change the LTOP enable temperature, (6) modify TS 2.10.1 to exactly specify the reactor coolant system (RCS) temperature at which the reactor can be made critical, and (7) add a TS for a maximum pressure value for the safety injection tanks. The use of a PTLR requires the relocation of TS Figure 2-1 (RCS Pressure—Temperature Limits for Heatup, Cooldown, and In-service Test) into Figure 5-1 of the PTLR. As a result of these changes, the following TSs are required to either be modified or added: define the PTLR in Definitions; TS 2.1.1(8); TS 2.1.1(11); Basis Section of TS 2.1.1; TS 2.1.2, including the TS 2.1.2 Basis and Reference Sections; TS 2.1.6(4); TS 2.3(1)(c); TS 2.3(3); TS 2.3 References; TS 2.10.1 and TS 2.10.1 Basis Section; Table 3-5, item 23, TS 3.3(1)(c); and TS 5.9.6.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The proposed changes do not involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed changes will not increase the probability or consequence of any accident for the following reasons:

(1) The proposed changes relocate the Pressure—Temperature (P-T) limit curves and low temperature over pressure protection (LTOP) system setpoints to the Pressure and Temperature Limits Report (PTLR). Compliance with these curves and limits continues to be required by the Technical Specifications (TSs). Changes to the curves will be controlled by TS 5.9.6, which contains the NRC approved methodologies used in the development of the PTLR. The change to the P-T limit curve as shown on Figure 5-1 of the PTLR is in compliance with Reference 10.11 [of the licensee's October 8, 2002, submittal], Westinghouse Electric Company/Combustion Engineering's (W/CE's) methodology and ASME Code Case N-640 for performing P-T limit curves.

(2) Revisions to the LTOP system limits can only be made in accordance with the approved methodologies stated in TS 5.9.6 with any resulting setpoint changes controlled by the 10 CFR 50.59 process. The PTLR in combination with the limitations imposed by the TSs will ensure the integrity of the reactor vessel pressure boundary.

(3) The conservative, but lower minimum boltup temperature and LTOP enable temperature are in compliance with Reference 10.12 [of the licensee's October 8, 2002, submittal]. Since the P-T limit curves and LTOP analysis are analyzed to the same temperatures as these proposed temperature values, there is no reduction to the margin of safety.

(4) Restricting the RCS temperature at which the reactor can be made critical is more conservative than the minimum temperature requirements for core critical operations based on fracture mechanics considerations as required by Reference 10.11 [of the licensee's October 8, 2002, submittal] during physics testing.

(5) Addition of a maximum pressure to the safety injection tanks (SITs) ensures compliance with Criterion 2 of 10 CFR 50.36(c)(2)(ii).

Therefore, the probability or consequence of any accident is not increased.

2. The proposed changes do not create the possibility of a new or different kind of accident from any accident previously evaluated.

The proposed revision does not change any equipment required to mitigate the consequences of an accident. The continued use of the same TS administrative controls prevents the possibility of a new or different kind of accident. Since the proposed changes do not involve the addition or modification of equipment nor alter the design of plant systems, the proposed changes do not create the possibility of a new or different kind of accident from any accident previously evaluated. The changes proposed do not change how design basis accident events are postulated nor do the changes themselves initiate a new kind of accident or failure mode with a unique set of conditions (proposed administrative controls). Therefore, the proposed change does not create the possibility of a new or different kind of accident from any previously evaluated.

3. The proposed changes do not involve a significant reduction in a margin of safety.

Relocating the P-T limit curves and LTOP system setpoints to the PTLR is in compliance with Reference 10.7 [of the licensee's October 8, 2002, submittal]. Future updates of the PTLR will be conducted under the 10 CFR 50.59 process utilizing NRC approved methodologies. Updating the P-T limit curve is in accordance with Reference 10.11 [of the licensee's October 8, 2002, submittal], W/CE's methodology and ASME Code Case N-640. Reduction of the minimum boltup temperature and LTOP enable temperature is in compliance with Reference 10.12 [of the licensee's October 8, 2002, submittal]. Restricting the reactor coolant system (RCS) temperature at which the reactor can be made critical is more conservative than the minimum temperature

requirements for core critical operations based on fracture mechanics considerations as required by Reference 10.11 [of the licensee's October 8, 2002, submittal], during physics testing. Addition of a maximum pressure to the SITs is in accordance with Criterion 2 of 10 CFR 50.36(c)(2)(ii). Additionally, the LTOP methodology and analysis conforms to Reference 10.10 [of the licensee's October 8, 2002, submittal]. Therefore, the proposed changes do not involve a significant reduction to the margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

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Pacific Gas and Electric Company, Docket Nos. 50-275 and 50-323, Diablo Canyon Nuclear Power Plant, Unit Nos. 1 and 2, San Luis Obispo County, California

Date of amendment requests: April 2, 2003.

Description of amendment requests: The proposed license amendments would revise Technical Specification (TS) 5.5.11, "Ventilation Filter Testing Program (VFTP)," to change the surveillance frequency, penetration, and relative humidity requirements for laboratory testing of the charcoal adsorber for the control room, auxiliary building, and fuel handling building ventilation systems. This would also eliminate the charcoal preheater testing requirements. TS 3.7.10, "Control Room Ventilation System (CRVS)," and TS 3.7.12, "Auxiliary Building Ventilation System (ABVS)," will also be revised to be consistent with these changes. These changes are in accordance with Regulatory Guide 1.52, Revision 3, "Design, Inspection, and Testing Criteria for Air Filtration and Adsorption Units of Post Accident Engineered-Safety-Feature Atmosphere Cleanup Systems in Light-Water-Cooled Nuclear Power Plants," Generic Letter 99-02, "Laboratory Testing of Nuclear-Grade Activated Charcoal," and the requirements in American Society for Testing and Materials D3803-1989, "Standard Technical Method for Nuclear-Grade Activated Carbon." In addition, TS 3.7.10 would be revised by adding a note allowing the control room boundary to be open intermittently under administrative control; adding a new required TS Action for two CRVS