

Trowbridge, 2300 N Street, NW.,
Washington, DC 20037.

NRC Section Chief: Stephen Dembek.

Union Electric Company, Docket No. 50-483, Callaway Plant, Unit 1, Callaway County, Missouri

Date of application request: June 27, 2003.

Description of amendment request:

The licensee is proposing to amend the operating license for the Callaway Plant to allow plant modifications in order to facilitate maintenance on the replacement steam generators (SGs) to be installed in Refueling Outage (RO) 14 (Fall 2005). The proposed modifications (1) replace the existing sludge lance platforms with new platforms to provide a larger platform area around each SG, and (2) cut a permanent access opening through the secondary shield wall to improve access to the sludge lance platforms. They are to be done in RO 13 (Spring 2004). Dynamic effects associated with large reactor coolant system (RCS) branch line ruptures are to be excluded using a proposed leak-before-break (LBB) methodology. The amendment would authorize changes to the Callaway licensing basis to be added to the Callaway Final Safety Analysis Report (FSAR). There are no proposed changes to the Technical Specifications. Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

Overall protection system performance will remain within the bounds of the previously performed accident analyses. The design of the protection systems will be unaffected. The reactor protection system and engineered safety feature actuation system will continue to function in a manner consistent with the plant design basis. All design, material, and construction standards that were applicable prior to the request are maintained.

Neither the currently intact "C" SG cubicle secondary shield wall, nor the proposed configuration that provides a permanent access opening, create accident initiation mechanisms that would increase the probability of an accident. There will be no change to normal plant operating parameters or accident mitigation performance.

The proposed amendment will not alter any assumptions or change any mitigation actions in the radiological consequence evaluations in the FSAR.

Therefore, the proposed amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. The proposed change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

There are no changes in the method by which any safety-related plant system performs its safety function. This amendment will not affect the normal method of power operation or change any operating parameters. No performance requirements will be affected, but SG maintenance access will be greatly improved.

No new accident scenarios, transient precursors, failure mechanisms, or limiting single failures are introduced as a result of this amendment. There will be no adverse effect or challenges imposed on any safety-related system as a result of this amendment.

Presence of a permanent access opening in the "C" loop SG secondary shield wall does not, of itself, create the possibility of a new accident since the secondary shield walls are not used for missile protection and the high-energy line breaks (greater than 10-inches in diameter) that would generate missiles will be removed from the structural design basis after NRC's review and acceptance of the LBB topical reports.

The proposed amendment does not alter the design or performance of the 7300 Process Protection System, Nuclear Instrumentation System, or Solid State Protection System used in the plant protection systems.

Therefore, the proposed amendment does not create the possibility of a new or different kind of accident from any previously evaluated.

3. The proposed change does not involve a significant reduction in a margin of safety.

There will be no effect on the manner in which safety limits or limiting safety system settings are determined nor will there be any effect on those plant systems necessary to assure the accomplishment of protection functions. There will be no impact on the overpower limit, departure from nucleate boiling ratio (DNBR) limits, heat flux hot channel factor (F_Q), nuclear enthalpy rise hot channel factor (FAH), loss-of-[off]-coolant accident peak cladding temperature (LOCA PCT), or peak local power density. The LBB margins discussed in NUREG-1061 Volume 3 are satisfied. The radiological dose consequence acceptance criteria listed in the [NRC] Standard Review Plan will continue to be met. The secondary shield walls are not fission product barriers. They provide radiation shielding to maintain occupational exposure ALARA [as low as is reasonably achievable] and provide structural support to primary coolant SSCs [structures, systems, and components].

The proposed amendment does not eliminate any surveillances or alter the Frequency of surveillances required by the Technical Specifications. The nominal Reactor Trip System (RTS) and Engineered Safety Features Actuation System (ESFAS) trip setpoints (TS Bases Tables B 3.3.1-1 and B 3.3.2-1), RTS and ESFAS allowable values (TS Tables 3.3.1-1 and 3.3.2-1), and the safety analysis limits assumed in the transient and accident analyses (FSAR Table 15.0-4) are unchanged. None of the acceptance criteria for any accident analysis is changed.

Therefore, the proposed amendment does not involve a significant reduction in a margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

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NRC Section Chief: Stephen Dembek.

Virginia Electric and Power Company, Docket Nos. 50-338 and 50-339, North Anna Power Station, Units No. 1 and 2, Louisa County, Virginia

Date of amendment request: March 28, 2002, as supplemented by letters dated May 13, June 19, and November 15, 2002, and May 6, May 9, May 27, and June 11 (2 letters), 2003. This notice supersedes the notice that was published on May 14, 2002 (67 FR 34496).

Description of amendment request:

The proposed amendments would permit Virginia Electric and Power Company to replace the existing Westinghouse fuel with Framatome ANP Advanced Mark-BW fuel at North Anna Power Station, Units 1 and 2. This submittal was accompanied by requested exemptions from the requirements of 10 CFR 50.44 and 10 CFR 50.46. These exemptions will be processed separately.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The probability of occurrence or the consequences of an accident previously evaluated is not significantly increased.

The proposed methodology has been generically reviewed and approved for use by the NRC for determining core operating limits prior to its use by Dominion. Analyzed events are assumed to be initiated by the failure of plant structures, systems, or components. The core operating limits developed in accordance with the new methodologies will be bounded by any limitations in the NRC safety evaluation report (SER) for the new methodologies. Application of the topical reports associated with the new methodologies will demonstrate that the integrity of the fuel will be maintained during normal operations and that design requirements will continue to be met. The proposed changes do not involve physical changes to any plant structure, system, or component. Therefore, the