

probability of occurrence of any accident previously evaluated is not significantly increased.

The consequences of a previously analyzed accident are dependent on the initial conditions assumed for the analysis, the behavior of the fuel during the analyzed accident, the availability and successful functioning of the equipment assumed to operate in response to the analyzed event, and the setpoints at which these actions are initiated. The proposed changes do not affect the performance of any equipment used to mitigate the consequences of an analyzed accident. As a result, no analysis assumptions are violated and there are no adverse effects on the factors that contribute to offsite or onsite dose resulting from an accident. The proposed changes do not affect setpoints that initiate protective or mitigative actions. The proposed changes ensure that plant structures, systems, and components are maintained consistent with the safety analysis and licensing basis. Based on this evaluation, there is no significant increase in the consequences of a previously analyzed event.

2. The possibility for a new or different type of accident from any accident previously evaluated is not created.

The proposed changes do not involve any physical alteration of plant systems, structures, or components, other than allowing for fuel design in accordance with NRC-approved methodologies. The proposed methodologies continue to meet applicable criteria for LBLOCA [large-break loss-of-coolant accident] and SBLOCA [small-break loss-of-coolant accident] analyses. No new or different equipment is being installed. No installed equipment is being operated in a different manner. There is no alteration to the parameters within which the plant is normally operated or in the setpoints that initiate protective or mitigative actions. As a result, no new failure modes are being introduced. There are no changes in the methods governing normal plant operation, nor are the methods utilized in response to plant transients changed. Therefore, the possibility for a new or different kind of accident from any accident previously evaluated is not created.

3. The margin of safety is not significantly reduced.

The margin of safety is established through the design of the plant structures, systems, and components, through the parameters within which the plant is operated, through the establishment of setpoints for the actuation of equipment relied upon to respond to an event, and through margins contained within safety analyses. The proposed changes in the methodologies used in the LBLOCA and SBLOCA analyses do not impact the condition or performance of structures, systems, setpoints, and components relied upon for accident mitigation. The proposed changes in the analysis methodologies comply with the requirements of 10 CFR 50.46 paragraph (a)(1)(i) (*i.e.*, not exceeding a peak cladding temperature of 2200°F for [SB] LOCA and a high probability that peak cladding temperature will remain below 2200°F for [LB] LOCA). Therefore, the margin of safety

as defined in the Bases to the North Anna Units 1 and 2 Technical Specifications is not significantly reduced.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

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NRC Section Chief: John A. Nakoski.

Notice of Issuance of Amendments to Facility Operating Licenses

During the period since publication of the last biweekly notice, the Commission has issued the following amendments. The Commission has determined for each of these amendments that the application complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment.

Notice of Consideration of Issuance of Amendment to Facility Operating License, Proposed No Significant Hazards Consideration Determination, and Opportunity for A Hearing in connection with these actions was published in the **Federal Register** as indicated.

Unless otherwise indicated, the Commission has determined that these amendments satisfy the criteria for categorical exclusion in accordance with 10 CFR 51.22. Therefore, pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared for these amendments. If the Commission has prepared an environmental assessment under the special circumstances provision in 10 CFR 51.12(b) and has made a determination based on that assessment, it is so indicated.

For further details with respect to the action see (1) the applications for amendment, (2) the amendment, and (3) the Commission's related letter, Safety Evaluation and/or Environmental Assessment as indicated. All of these items are available for public inspection at the Commission's Public Document Room, located at One White Flint North, Public File Area 01F21, 11555 Rockville Pike (first floor), Rockville, Maryland.

Publicly available records will be accessible from the Agencywide Documents Access and Management Systems (ADAMS) Public Electronic Reading Room on the internet at the NRC web site, <http://www.nrc.gov/reading-rm/adams.html>. If you do not have access to ADAMS or if there are problems in accessing the documents located in ADAMS, contact the NRC Public Document Room (PDR) Reference staff at 1-800-397-4209, 301-415-4737 or by email to pdr@nrc.gov.

Arizona Public Service Company, et al., Docket Nos. STN 50-528, STN 50-529, and STN 50-530, Palo Verde Nuclear Generating Station, Units Nos. 1, 2, and 3, Maricopa County, Arizona

Date of application for amendments: April 15, 2003.

Brief description of amendments: The amendments revise Sections 2.2, "SL [Safety Limits] Violations," for reporting such violations to positions in the plant organization; 5.2.1, "Onsite and Offsite Organization," for the position responsible for overall safe plant operation; and 5.5.1, "Offsite Dose Calculation Manual (ODCM)," to replace the positions of Vice President, Nuclear Production, and Director, Site Chemistry, with other positions in the plant organization. Also, there would be the format change of adding the title of Section 2.2 near the top of TS page 2.0-2.

Date of issuance: June 26, 2003.

Effective date: June 26, 2003, and shall be implemented within 30 days of the date of issuance.

Amendment Nos.: Unit 1-146, Unit 2-146, Unit 3-146.

Facility Operating License Nos. NPF-41, NPF-51, and NPF-74: The amendments revised the Technical Specifications.

Date of initial notice in Federal Register: May 27, 2003 (68 FR 28845).

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated June 26, 2003.

No significant hazards consideration comments received: No.

Duke Energy Corporation, et al., Docket Nos. 50-413 and 50-414, Catawba Nuclear Station, Units 1 and 2, York County, South Carolina

Date of application for amendments: August 26, 2002, as supplemented by letter dated June 18, 2003.

Brief description of amendments: The amendments revise the Technical Specifications regarding the Diesel Fuel Oil Testing Program.

Date of issuance: July 10, 2003.

Effective date: As of the date of issuance and shall be implemented