

" K_{IA} fracture toughness curve," as defined in ASME Code, section XI, appendices A and G, respectively) for reactor vessel materials in determining the P-T limits for heatup, cooldown, and inservice testing.

The proposed action is in accordance with the licensee's application dated October 11, 2002.

The Need for the Proposed Action

The provisions of ASME Code Case N-641 were incorporated in appendix G of section XI of the ASME Code in the 1998 though the 2000 Addenda, which is the edition and addenda of record in the 2003 Edition of 10 CFR part 50. However, the proposed action is needed to apply Code Case N-641, because the Seabrook licensing basis has only been updated to include the 1995 Edition through the 1996 Addenda of the ASME Code.

Environmental Impacts of the Proposed Action

The NRC has completed its evaluation of the proposed action and concludes that, as set forth below, there are no significant environmental impacts associated with the use of ASME Code Case N-641 in developing RPV P-T limits for heatup, cooldown, and inservice testing. The proposed action does not adversely affect the integrity of the reactor vessel or the function of the reactor vessel to act as a radiological barrier during an accident.

The proposed action will not significantly increase the probability or consequences of accidents, no changes are being made in the types of effluents that may be released offsite, and there is no significant increase in occupational or public radiation exposure. Therefore, there are no significant radiological environmental impacts associated with the proposed action.

With regard to potential non-radiological impacts, the proposed action does not have a potential to affect any historic sites. The proposed action does not affect non-radiological plant effluents and has no other environmental impact. Therefore, there are no significant non-radiological environmental impacts associated with the proposed action.

Accordingly, the NRC concludes that there are no significant environmental impacts associated with the proposed action.

Environmental Impacts of the Alternatives to the Proposed Action

As an alternative to the proposed action, the staff considered denial of the proposed action (*i.e.*, the "no-action"

alternative). Denial of the application would result in no change in current environmental impacts. The environmental impacts of the proposed action and the alternative action are similar.

Alternative Use of Resources

The action does not involve the use of any different resources than those previously considered in the Final Environmental Statement for the Seabrook Station, Unit No. 1, dated December 1982.

Agencies and Persons Consulted

On June 4, 2003, the staff consulted with the New Hampshire State Official, Mike Nawoj of the New Hampshire Office of Emergency Management, and with the Massachusetts State Official, Diane Brown-Couture, of the Massachusetts Emergency Management Agency, regarding the environmental impact of the proposed action. The State Officials had no comments.

Finding of No Significant Impact

On the basis of the Environmental Assessment, the NRC concludes that the proposed action will not have a significant effect on the quality of the human environment. Accordingly, the NRC has determined not to prepare an environmental impact statement for the proposed action.

For further details with respect to the proposed action, see the licensee's letter dated October 11, 2002. Documents may be examined, and/or copied for a fee, at the NRC's Public Document Room (PDR), located at One White Flint North, 11555 Rockville Pike (first floor), Rockville, Maryland. Publicly available records will be accessible electronically from the Agencywide Documents Access and Management System (ADAMS) Public Electronic Reading Room on the Internet at the NRC Web site, <http://www.nrc.gov/reading-rm/adams.html>. Persons who do not have access to ADAMS, or who encounter problems in accessing the documents located in ADAMS, should contact the NRC PDR Reference staff by telephone at 1-800-397-4209 or 301-415-4737, or by e-mail to pdr@nrc.gov.

Dated at Rockville, Maryland, this 21st day of July, 2003.

For the Nuclear Regulatory Commission.

James W. Clifford,

Chief, Section 2, Project Directorate I, Division of Licensing Project Management, Office of Nuclear Reactor Regulation.

[FR Doc. 03-18962 Filed 7-24-03; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

[Docket No. 50-285]

Omaha Public Power District; Fort Calhoun Station, Unit 1; Environmental Assessment and Finding of No Significant Impact

The U.S. Nuclear Regulatory Commission (NRC) is considering issuance of an exemption from Title 10 of the Code of Federal Regulations (10 CFR) Part 50, Appendix G for Facility Operating License No. DPR-40, issued to Omaha Public Power District (the licensee), for operation of the Fort Calhoun Station, Unit No. 1 (FCS), located in Washington County, Nebraska. Therefore, as required by 10 CFR 51.21, the NRC is issuing this environmental assessment and finding of no significant impact.

Environmental Assessment

Identification of the Proposed Action

The proposed action would exempt the licensee from certain requirements of Appendix G to 10 CFR Part 50 to allow the application of the methodology in Combustion Engineering (CE) Topical Report NPSD-683-A, Revision 6, "Development of a RCS Pressure and Temperature Limits Report for the Removal of P-T Limits and LTOP Requirements from the Technical Specifications," for the calculation of flaw stress intensity factors due to thermal stress loadings (K_{It}).

The proposed action is in accordance with the licensee's application dated October 8, 2002.

The Need for the Proposed Action

In the associated exemption, the staff has determined that, pursuant to 10 CFR 50.12(a)(2)(ii), the underlying purpose of the regulation will continue to be served by the implementation of the alternative methodology. The proposed action would revise the currently-approved methodology for pressure temperature (P-T) limit calculations to incorporate the methodology approved for use in CE NPSD-683-A, Revision 6. CE NPSD-683-A, Revision 6, allows the use of an alternate methodology to calculate the flaw stress intensity factors due to thermal stress loadings (K_{It}). The exemption is needed because the methodology in CE NPSD-683-A, Revision 6, could not be shown to be conservative with respect to the methodology for the determination of K_{It} provided in Editions and Addenda of ASME Code, Section XI, Appendix G, through the 1995 Edition and 1996