

Date of initial notice in Federal Register: October 1, 2002 (67 FR 61679). The May 9, 2003, supplemental letter provided clarifying information that did not change the scope of the original **Federal Register** notice or the original no significant hazards consideration determination. The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated July 24, 2003.

No significant hazards consideration comments received: No.

Entergy Nuclear Operations, Inc., Docket No. 50-293, Pilgrim Nuclear Power Station, Plymouth County, Massachusetts

Date of application for amendment: July 5, 2002, as supplemented August 13, 2002.

Brief description of amendment: The amendment relocates portions of Technical Specification (TS) 3/4.6.B, "Primary System Boundary—Coolant Chemistry," from the TSs to the Updated Final Safety Analysis Report (UFSAR). The portions of the TSs relocated to the UFSAR are the reactor coolant chemistry requirements for conductivity and chloride concentration. Specifically, TSs 3/4.6.B.2, 3/4.6.B.3, and 3.6.B.4 are relocated to the UFSAR.

Date of issuance: July 21, 2003.

Effective date: As of the date of issuance, and shall be implemented within 60 days.

Amendment No.: 202.

Facility Operating License No. DPR-35: Amendment revised the Technical Specifications.

Date of initial notice in Federal Register: May 27, 2003 (68 FR 28850). The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated July 21, 2003.

No significant hazards consideration comments received: No.

FirstEnergy Nuclear Operating Company, et al., Docket Nos. 50-334 and 50-412, Beaver Valley Power Station, Unit Nos. 1 and 2 (BVPS-1 and 2), Beaver County, Pennsylvania

Date of application for amendments: October 31, 2001, as supplemented by letters dated December 21, 2001, and February 4, May 31, and December 2, 2002.

Brief description of amendments: The changes relocated the pressure temperature (P/T) limit curves and low temperature overpressure protection system limits to the Pressure and Temperature Limits Report (PTLR) in the BVPS-1 and 2 Licensing Requirements Manual and the reference that report in the affected TS limiting

conditions for operation and Bases. The changes also included the addition of the PTLR to the Definitions Section of the TSs and added a new section to the reporting requirements in the Administrative Controls Section of the TSs delineating the necessary reports. The proposed changes were based on Generic Letter 96-03, "Relocation of the Pressure Temperature Limit Curves and Low Temperature Overpressure Protection System Limits," dated January 31, 1996, and the Nuclear Regulatory Commission (NRC) staff's approval of the BVPS-1 and 2 plant-specific P/T limits methodology documented in the letter from Richard J. Laufer, NRC, to Mark B. Bezilla, FENOC, dated October 8, 2002.

Date of issuance: July 15, 2003.

Effective date: Effective as of the date of issuance and shall be implemented within 60 days.

Amendment Nos.: 256 and 138.

Facility Operating License Nos. DPR-66 and NPF-73: Amendments revised the Technical Specifications.

Date of initial notice in Federal Register: December 26, 2001 (66 FR 66465). The supplements dated December 21, 2001, and February 4, May 31, and December 2, 2002, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the NRC staff's original proposed no significant hazards consideration determination. The Commission's related evaluation of the amendments is contained in a Safety Evaluation dated July 15, 2003.

No significant hazards consideration comments received: No.

Florida Power Corporation, et al., Docket No. 50-302, Crystal River Unit No. 3 Nuclear Generating Plant, Citrus County, Florida

Date of application for amendment: October 11, 2002, as supplemented March 4, 2003.

Brief description of amendment: The amendment revises Crystal River Unit 3 Improved Technical Specifications (ITS) 3.3.15, "Reactor Building Purge Isolation-High Radiation"; ITS Bases 3.7.15, "Spent Fuel Assembly Storage"; ITS 3.9.3, "Containment Penetrations"; and ITS 3.9.6, "Refueling Canal Water Level" to account for the handling of irradiated fuel within containment that has not occupied part of a critical reactor core within the previous 72 hours.

Date of issuance: July 14, 2003.

Effective date: As of the date of issuance, and shall be implemented prior to entering Mode 6 for the Cycle 13 refueling outage.

Amendment No.: 208.

Facility Operating License No. DPR-72: Amendment revised the Technical Specifications.

Date of initial notice in Federal Register: February 4, 2003 (68 FR 5676). The March 4, 2003, supplement contained clarifying information only and did not change the initial no significant hazards consideration determination or expand the scope of the initial application. The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated July 14, 2003.

No significant hazards consideration comments received: No.

Indiana Michigan Power Company, Docket No. 50-315, Donald C. Cook Nuclear Plant, Unit 1, Berrien County, Michigan

Date of application for amendment: December 10, 2002.

Brief description of amendment: The amendment consists of changes to the Donald C. Cook Nuclear Plant (D. C. Cook) Unit 1 Technical Specifications related to the reactor pressure vessel (RPV) operating limits at low temperatures. The amendment approves revised pressure-temperature limits for the RPV to be applicable for a maximum of 32 effective full-power years of facility operation. These changes were based, in part, on the use of American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code (Code) Case N-641.

Date of issuance: July 18, 2003.

Effective date: As of the date of issuance and shall be implemented prior to startup from Unit 1 refueling outage 19.

Amendment No.: 278.

Facility Operating License No. DPR-58: Amendment revises the Technical Specifications.

Date of initial notice in Federal Register: April 1, 2003 (68 FR 15762). The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated July 18, 2003.

No significant hazards consideration comments received: No.

Maine Yankee Atomic Power Company, Docket No. 50-309, Maine Yankee Atomic Power Station, Lincoln County, Maine

Date of application for amendment: April 24, 2003.

Brief description of amendment: The amendment revises the Technical Specifications to eliminate the requirement for at least one person qualified to stand watch being present in the control room when irradiated fuel is stored in the fuel storage pool.