

Attorney for licensee: Ms. Lisa F. Vaughn, Duke Energy Corporation, 422 South Church Street, Charlotte, North Carolina 28201-1006.

NRC Section Chief: John A. Nakoski.

Duke Energy Corporation, Docket Nos. 50-269, 50-270, and 50-287, Oconee Nuclear Station, Units 1, 2, and 3, Oconee County, South Carolina

Date of amendment request: July 10, 2003.

Description of amendment request:

The proposed amendments would revise the Technical Specifications to eliminate requirements that are no longer applicable due to the completion of the automatic feedwater system modifications.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

Pursuant to 10 CFR 50.91, Duke Energy Corporation (Duke) has made the determination that this amendment request involves a No Significant Hazards Consideration by applying the standards established by the NRC regulations in 10 CFR 50.92. This ensures that operation of the facility in accordance with the proposed amendment would not:

(1) Involve a significant increase in the probability or consequences of an accident previously evaluated: The proposed change to the Oconee Technical Specifications removes obsolete requirements associated with the Main Steam Line Break (MSLB) detection circuitry that are no longer necessary because of the completion of the Automatic Feedwater Isolation System (AFIS) modification on all three Oconee Units. AFIS replaced the MSLB detection system. As such, the proposed change is administrative. No actual plant equipment, operating practices, or accident analyses are affected by this change. Therefore, the proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

(2) Create the possibility of a new or different kind of accident from any kind of accident previously evaluated: The proposed change to the Oconee Technical Specifications removes obsolete requirements associated with the MSLB detection circuitry that are no longer necessary because of the completion of the AFIS modification on all three Oconee Units. AFIS replaced the MSLB detection system. As such, the proposed change is administrative. No actual plant equipment, operating practices, or accident analyses are affected by this change. No new accident causal mechanisms are created as a result of this change. The proposed change does not impact any plant systems that are accident initiators; neither does it adversely impact any accident mitigating systems. Therefore, this change does not create the

possibility of a new or different kind of accident from any accident previously evaluated.

(3) Involve a significant reduction in a margin of safety: The proposed change does not adversely affect any plant safety limits, set points, or design parameters. The change also does not adversely affect the fuel, fuel cladding, Reactor Coolant System, or containment integrity. The proposed change eliminates obsolete requirements and is administrative in nature. Therefore, the proposed change does not involve a reduction in a margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

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Exelon Generation Company, LLC, Docket Nos. 50-237 and 50-249, Dresden Nuclear Power Station, Units 2 and 3, Grundy County, Illinois and Docket Nos. 50-254 and 50-265, Quad Cities Nuclear Power Station, Units 1 and 2, Rock Island County, Illinois

Date of application for amendment request: October 10, 2002, as supplemented March 21 and March 28, 2003.

Description of amendment request:

The proposed amendment revises the licensing bases and Technical Specifications by utilizing an alternative source term in the design-basis radiological analyses in accordance with 10 CFR 50.67, with the exception that Technical Information Document 14844 will continue to be used as the radiation dose basis for equipment qualification.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The proposed changes do not involve a significant increase in the probability or consequences of an accident previously evaluated.

The implementation of alternative source term (AST) assumptions has been evaluated in revisions to the analyses of the following limiting design basis accidents at Dresden Nuclear Power Station (DNPS) and Quad Cities Nuclear Power Station (QCNPS):

Loss-of-Coolant Accident,
Main Steam Line Break Accident,
Fuel Handling Accident, and
Control Rod Drop Accident.

Based upon the results of these analyses, it has been demonstrated that, with the requested changes, the dose consequences of these limiting events is within the regulatory guidance provided by the NRC for use with the AST. This guidance is presented in 10 CFR 50.67 and associated Regulatory Guide 1.183, and Standard Review Plan Section 15.0.1.

Requirements for secondary containment operability, secondary containment isolation valves, the Standby Gas Treatment (SGT) System, the Control Room Emergency Ventilation (CREV) System, and the Control Room Emergency Ventilation Air Conditioning (AC) System during movement of irradiated fuel assemblies that have decayed at least 24-hours and during core alterations are being eliminated. This is acceptable because, with the application of AST, none of these systems are credited in mitigating the consequences of a fuel handling accident after a 24-hour decay period.

The proposed change also increases the maximum allowable primary containment leakage and the maximum allowable main steam isolation valve leakage limits. This is acceptable due to the new assumptions, used in calculating control room and offsite dose following a design basis loss-of-coolant accident, related to application of AST.

The proposed changes do not affect the design or operation of the facility; rather, once the occurrence of an accident has been postulated, the new source term is an input to evaluate the consequence. The radiological consequences of the above design basis accidents have been evaluated with application of AST assumptions. The results conclude that the radiological consequences remain within applicable regulatory limits. Therefore, the proposed changes do not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. The proposed changes do not create the possibility of a new or different kind of accident from any accident previously evaluated.

The application of AST does not affect the design, functional performance or operation of the facility. Similarly, it does not affect the design or operation of any structures, systems or components involved in the mitigation of any accidents, nor does it affect the design or operation of any component in the facility such that new equipment failure modes are created.

As such the proposed amendment will not create the possibility of a new or different kind of accident from any accident previously evaluated.

3. The proposed change does not involve a significant reduction in a margin of safety.

Approval of the basis change from the original source term developed in accordance with Technical Information Document (TID) 14844 to a new AST, as described in Regulatory Guide 1.183, is requested. The results of the accident analyses revised in support of the proposed changes, and the requested Technical Specification changes, are subject to revised acceptance criteria. These analyses have been performed using conservative methodologies.