

or different kind of accident from those previously analyzed has not been created.

3. The proposed amendment will not involve a significant reduction in the margin of safety.

This license amendment request proposes to add filter test face velocity minimum values for the control room special ventilation system, auxiliary building special ventilation system, spent fuel pool special and inservice purge ventilation system and shield building ventilation system. These additional Technical Specification limits on system performance assures these ventilation systems are tested and maintained within their designed function limits and may increase the margin of safety for these systems. Therefore this change does not involve a significant reduction in the margin of safety.

Editorial and administrative changes

1. The proposed amendment will not involve a significant increase in the probability or consequences of an accident previously evaluated.

This license amendment request proposes editorial changes to Technical Specification Section 5.5.9, including replacement of ventilation system names with abbreviations and miscellaneous changes associated with addition of a new paragraph to this section, and proposes an administrative change to delete the Operating License Additional Condition for each unit that relates to NRC Generic Letter 99-02. Since these changes are editorial or administrative, they do not change any plant operating limits or technical requirements. Therefore these changes do not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. The proposed amendment will not create the possibility of a new or different kind of accident from any accident previously analyzed.

This proposed change does not alter the design, function, or operation of any plant component and does not install any new or different equipment. The malfunction of safety related equipment, assumed to be operable in the accident analyses, would not be caused as a result of the proposed technical specification change. No new failure mode has been created and no new equipment performance burdens are imposed. Therefore, the possibility of a new or different kind of accident from those previously analyzed has not been created.

3. The proposed amendment will not involve a significant reduction in the margin of safety.

This license amendment request proposes editorial changes to Technical Specification Section 5.5.9, including replacement of ventilation system names with abbreviations and miscellaneous changes associated with addition of a new paragraph to this section, and proposes an administrative change to delete the Operating License Additional Condition for each unit that relates to NRC Generic Letter 99-02. Since these changes are editorial or administrative, they do not change any plant operating limits or technical requirements. Therefore these changes do not involve a significant reduction in the margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment requests involve no significant hazards consideration.

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NRC Section Chief: L. Raghavan.

Omaha Public Power District, Docket No. 50-285, Fort Calhoun Station, Unit No. 1, Washington County, Nebraska

Date of amendment request: July 25, 2003.

Description of amendment request:

The proposed amendment modifies Technical Specification 2.1.4, "Reactor Coolant System (RCS) Leakage Limits." The proposed amendment will: (1) Add a requirement for no RCS pressure boundary leakage, (2) combine the existing RCS leakage limits into a format similar to the Improved Standard Technical Specification (ISTS), and (3) replace the existing basis associated with this specification with a basis similar in format and content of the ISTS. The proposed changes will assure that the design criteria of no RCS pressure boundary leakage is maintained and bring the Fort Calhoun Station, Unit 1 (FCS) RCS leakage specifications into alignment with the Improved Standard Technical Specifications. This amendment is modeled after the Improved Standard Technical Specifications.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed changes to Technical Specifications 2.1.4 establish a limit on reactor coolant system pressure boundary leakage and provide an allowed outage time and actions required for restoring operability. The proposed Technical Specifications address the regulatory requirements for equipment required for FCS Design Criterion 16 (similar to 10 CFR 50 GDC [General Design Criterion] 30). The change will ensure that proper Limiting Conditions for Operation are entered for equipment or functional inoperability. There are no physical alterations being made to the reactor coolant system or related systems. Therefore, the proposed changes do not involve a significant increase in the probability or

consequences of an accident previously evaluated.

2. The proposed change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

The proposed changes will not result in any physical alterations to the reactor coolant system, any plant configuration, systems, equipment, or operational characteristics. There will be no changes in operating modes, or safety limits, or instrument limits. With the proposed changes in place, Technical Specifications will retain requirements for the reactor coolant system. Therefore, the proposed changes do not create the possibility of a new or different kind of accident from any previously evaluated.

3. The proposed change does not involve a significant reduction in a margin of safety.

The proposed changes clarify the regulatory requirements for the reactor coolant system as defined by FCS Design Criterion 16 (similar to 10 CFR 50 GDC 30). The times established are within those invoked by the present Technical Specifications or equal to those previously reviewed and approved for use by the NRC. The proposed changes will not alter any physical or operational characteristics of the reactor coolant system and associated systems and equipment. Therefore, the proposed changes do not involve a reduction in a margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

Attorney for licensee: James R. Curtiss, Esq., Winston & Strawn, 1400 L Street, NW, Washington, DC 20005-3502.

NRC Section Chief: Stephen Dembek.

Omaha Public Power District, Docket No. 50-285, Fort Calhoun Station, Unit No. 1, Washington County, Nebraska

Date of amendment request: July 25, 2003.

Description of amendment request:

The proposed amendment modifies Technical Specifications (TS) 3.0.2, Table 3-2, Table 3-5, 3.6, 3.7, 3.8, and the Definitions Section. This proposed change provides a risk-informed alternative to the existing surveillance interval for the integrated engineered safety features (ESF) and loss-of-offsite power (LOOP) testing required to be performed on each ESF equipment train each outage. The proposed change modifies the surveillance interval requirement for these refueling interval surveillance requirements to go to a staggered test-basis scheme. Using a staggered test basis, only one train would be tested each refueling outage. This amendment is modeled after the