

amendments revised the Technical Specifications.

Date of initial notice in Federal Register: June 10, 2003 (68 FR 34662). The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated August 13, 2003.

No significant hazards consideration comments received: No.

Entergy Nuclear Operations, Inc., Docket No. 50-003, Indian Point Nuclear Generating Station, Unit 1

Date of amendment request: May 30, 2002.

Brief description of amendment: It would revise the Indian Point Nuclear Generating Station, Unit 1 (IP1) Technical Specifications (TSs) to facilitate the Indian Point Generating Station, Unit 2 (IP2) transition to the Improved TSs. The amendment also revises the requirements of the "Order Approving Decommissioning Plan and Authorizing Decommissioning of Facility"¹ to ensure compliance with the current requirements of 10 CFR 50.59 and 10 CFR 50.83. It also revises the expiration date of Provisional Operating License No. DPR-5 for IP1 to be current with the expiration date for the Facility Operating License No. DPR-26 for IP2.

Date of issuance: August 11, 2003.

Effective date: As of the date of issuance to be implemented within 60 days from the date of issuance.

Amendment No.: 52.

Provisional Operating License No. DPR-5: The amendment revised the Technical Specifications, and made changes to and revised the expiration date for IP1 Provisional Operating License DPR-5.

Date of initial notice in Federal Register: July 9, 2002 (67 FR 45564).

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated August 11, 2003.

No significant hazards consideration comments received: No.

Entergy Operations, Inc., Docket No. 50-368, Arkansas Nuclear One, Unit No. 2, Pope County, Arkansas

Date of application for amendment: September 19, 2002, as supplemented by letters dated January 8, May 22, and July 1, 2003.

Brief description of amendment: The amendment extends the allowable outage time for the emergency diesel generators from 72 hours to a maximum of 14 days.

Date of issuance: August 8, 2003.

Effective date: As of the date of issuance to be implemented within 30 days from the date of issuance.

Amendment No.: 249.

Facility Operating License No. NPF-6: Amendment revised the Technical Specifications.

Date of initial notice in Federal Register: November 12, 2002 (67 FR 68733). The January 8, May 22, and July 1, 2003, supplemental letters provided clarifying information that did not change the scope of the original **Federal Register** notice or the original no significant hazards consideration determination.

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated August 8, 2003.

No significant hazards consideration comments received: No.

Exelon Generation Company, LLC, Docket Nos. STN 50-454 and STN 50-455, Byron Station, Unit Nos. 1 and 2, Ogle County, Illinois; Docket Nos. STN 50-456 and STN 50-457, Braidwood Station, Unit Nos. 1 and 2, Will County, Illinois

Date of application for amendments: August 7, 2002, as supplemented by your letters dated February 28, and May 27, 2003.

Brief description of amendments: The amendments revised the limiting condition for operation, the associated Conditions and Required Actions of Technical Specification (TS) 3.7.1, "Main Steam Safety Valves (MSSVs)," and the values in Table 3.7.1-1, "Operable Main Steam Safety Valves versus Applicable Power in Percent of Rated Thermal Power," by requiring five MSSVs per steam generator to be operable consistent with the accident analyses assumptions. The amendments also modify the associated Required Actions of TS 3.7.1 by adding a requirement to reduce the Power Range Neutron Flux-High reactor trip setpoint when one or more steam generators with one or more MSSVs are inoperable.

Date of issuance: August 12, 2003.

Effective date: As of the date of issuance and shall be implemented within 30 days.

Amendment Nos.: 133/133, 128/128.

Facility Operating License Nos. NPF-37, NPF-66, NPF-72 and NPF-77: The amendments revised the Technical Specifications.

Date of initial notice in Federal Register: October 1, 2002 (67 FR 61681). The supplemental letters contained clarifying information and did not change the initial no significant hazards consideration determination

and did not expand the scope of the original **Federal Register** notice.

The Commission's related evaluation of the amendments is contained in a Safety Evaluation dated August 12, 2003.

No significant hazards consideration comments received: No.

Exelon Generation Company, LLC, Docket Nos. 50-373 and 50-374, LaSalle County Station, Units 1 and 2, LaSalle County, Illinois

Date of application for amendments: December 20, 2002, as supplemented by letters dated May 30, and June 27, 2003.

Brief description of amendments: The amendments approve changes to the LaSalle County Station facility as described in the Updated Final Safety Analysis Report. The amendments modify the basis for compliance with the requirements of appendix H to title 10 of the Code of Federal Regulations part 50 (appendix H to 10 CFR part 50), "Reactor Vessel Material Surveillance Program Requirements," by approving implementation of the Boiling-Water Reactor Vessel and Internals Project reactor pressure vessel integrated surveillance program.

Date of issuance: August 13, 2003.

Effective date: As of the date of issuance and shall be implemented within 30 days.

Amendment Nos.: 160/146.

Facility Operating License Nos. NPF-11 and NPF-18: The amendments approve revisions to the Updated Final Safety Analysis Report.

Date of initial notice in Federal Register: February 4, 2003 (68 FR 5669). The supplemental letters contained clarifying information and did not change the initial no significant hazards consideration determination and did not expand the scope of the original **Federal Register** Notice.

The Commission's related evaluation of the amendments is contained in a Safety Evaluation dated August 13, 2003.

No significant hazards consideration comments received: No.

FirstEnergy Nuclear Operating Company, Docket No. 50-346, Davis-Besse Nuclear Power Station, Unit 1, Ottawa County, Ohio

Date of application for amendment: November 30, 2001.

Brief description of amendment: This amendment revises Technical Specification 3/4.4.4, "Reactor Coolant System—Pressurizer," to adopt a new pressurizer high-level limit and to revise the required action when the pressurizer is inoperable.

Date of issuance: August 12, 2003.

¹ NRC letter to Consolidated Edison, "Order to Authorize Decommissioning and Amendment No. 45 to License No. DPR-5 for Indian Point Unit 1 (TAC No. M59664)," dated January 31, 1996.