

The postulated primary-to-secondary leakage as an input parameter is used to analyze the potential radiological consequences of a MSLB accident. This postulated leakage occurs after an accident is initiated. As a result, the proposed leakage rate is not an initiator of any accident and no new failure modes are created. Exceeding the technical specification limits on reactor coolant system (RCS) operational leakage is not permitted.

The consequences of the MSLB are currently analyzed for a one gpm primary-to-secondary accident leakage in the faulted steam generator. Increasing the postulated accident leakage to three gpm increased the radiological consequences. This is a small increase in leakage which is not considered significant since the dose does not exceed the appropriate fraction of the 10 CFR Part 100, "Reactor Site Criteria," dose limits as specified in NUREG-0800, "Standard Review Plan," for an MSLB accident or the 10 CFR Part 50, Appendix A, General Design Criteria (GDC) 19, "Control Room," limits.

2. Does the proposed change create the possibility of a new or different kind of accident from any accident previously evaluated?

No. No new failure modes are created by the increase in the postulated primary-to-secondary leakage during an MSLB accident. The credible failure mode associated with this increase in leakage is for a steam generator tube to rupture during the MSLB accident. The use of the alternate repair criteria for axial ODSCC at the tube support plate has been previously approved for WBN. Under the alternate repair criteria for tubes that exhibit axial ODSCC at the tube support plate, a conditional burst probability calculation is performed to provide a conservative assessment of tube structural integrity during a postulated MSLB occurring at the end-of-cycle. The calculation is compared to a threshold value. If the burst probability calculation is greater than or equal to the threshold value, then tubes will be plugged to decrease below the threshold. This limits the probability of a steam generator tube rupture during an MSLB accident. Tubes that are outside of the alternate repair criteria will be plugged as specified in the WBN technical specification to maintain integrity. Additionally, RCS operational leakages are subject to continual surveillance and an accumulation of minor leaks which exceed the limits established in the technical specification is not permitted during unit operation. As previously stated, the postulated primary-to-secondary accident leakage is an input parameter and not an initiator of any accidents. The proposed increase in leakage has no significant effect on the configuration of the plant or the manner in which it is operated.

Therefore, the proposed change does not create the possibility of a new or different kind of accident from any previously evaluated.

3. Does the proposed change involve a significant reduction in a margin of safety?

No. An increase in the primary-to-secondary leakage during an MSLB accident allows more axial ODSCC affected tubes to remain in service; however, the structural

and leakage integrity of the tubes is assured by compliance with the alternate repair criteria. The affected tubes must meet specific conditions in order to remain in service. The tubes that remain in service as a result of the proposed increase in leakage must meet the requirements for determining the structural and leakage integrity. Tubes that are outside of the alternate repair criteria will be plugged as specified in the WBN technical specification to maintain integrity. The activity in the steam and power conversion system is continually monitored and an accumulation of minor leaks which exceed the limits established in the technical specification is not permitted during unit operation.

As specified in NUREG-0800, the dose mitigation features, in this case, leakage, are acceptable since the whole body and thyroid doses at the exclusion area and the lower population zone outer boundaries do not exceed the exposure guidelines. The control room doses do not exceed the requirements in 10 CFR Part 50, Appendix A, GDC 19.

Therefore, the proposed change does not involve a significant reduction in a margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

The Commission is seeking public comments on this proposed determination. Any comments received within 30 days after the date of publication of this notice will be considered in making any final determination.

Normally, the Commission will not issue the amendment until the expiration of the 30-day notice period. However, should circumstances change during the notice period such that failure to act in a timely way would result, for example, in derating or shutdown of the facility, the Commission may issue the license amendment before the expiration of the 30-day notice period, provided that its final determination is that the amendment involves no significant hazards consideration. The final determination will consider all public and State comments received. Should the Commission take this action, it will publish in the **Federal Register** a notice of issuance and provide for opportunity for a hearing after issuance. The Commission expects that the need to take this action will occur very infrequently.

Written comments may be submitted by mail to the Chief, Rules and Directives Branch, Division of Administrative Services, Office of Administration, U.S. Nuclear Regulatory

Commission, Washington, DC 20555-0001, and should cite the publication date and page number of this **Federal Register** notice. Written comments may also be delivered to Room 6D59, Two White Flint North, 11545 Rockville Pike, Rockville, Maryland, from 7:30 a.m. to 4:15 p.m. Federal workdays. Documents may be examined, and/or copied for a fee, at the NRC's Public Document Room, located at One White Flint North, Public File Area O1 F21, 11555 Rockville Pike (first floor), Rockville, Maryland.

The filing of requests for hearing and petitions for leave to intervene is discussed below.

By October 20, 2003, the licensee may file a request for a hearing with respect to issuance of the amendment to the subject facility operating license and any person whose interest may be affected by this proceeding and who wishes to participate as a party in the proceeding must file a written request for a hearing and a petition for leave to intervene. Requests for a hearing and a petition for leave to intervene shall be filed in accordance with the Commission's "Rules of Practice for Domestic Licensing Proceedings" in 10 CFR Part 2. Interested persons should consult a current copy of 10 CFR 2.714, which is available at the Commission's Public Document Room, located at One White Flint North, 11555 Rockville Pike (first floor), Rockville, Maryland, or electronically on the Internet at the NRC Web site <http://www.nrc.gov/reading-rm/doc-collections/cfr/>. If there are problems in accessing the document, contact the Public Document Room Reference staff at 1-800-397-4209, 301-415-4737, or by e-mail to [pdr@nrc.gov](mailto:pdr@nrc.gov). If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition; and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of hearing or an appropriate order.

As required by 10 CFR 2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding, and how that interest may be affected by the results of the proceeding. The petition should specifically explain the reasons why intervention should be permitted with particular reference to the following factors: (1) The nature of the petitioner's right under the Act to be made party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in