

leave to intervene and requests for hearing be transmitted to the Secretary of the Commission either by means of facsimile transmission to 301-415-1101 or by e-mail to hearingdocket@nrc.gov. A copy of the request for hearing and petition for leave to intervene should also be sent to the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and because of continuing disruptions in delivery of mail to United States Government offices, it is requested that copies be transmitted either by means of facsimile transmission to 301-415-3725 or by e-mail to OGCMailCenter@nrc.gov. A copy of the request for hearing and petition for leave to intervene should also be sent to the attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for a hearing will not be entertained absent a determination by the Commission, the presiding officer or the Atomic Safety and Licensing Board that the petition and/or request should be granted based upon a balancing of factors specified in 10 CFR 2.714(a)(1)(i)-(v) and 2.714(d).

For further details with respect to this action, see the application for amendment which is available for public inspection at the Commission's PDR, located at One White Flint North, Public File Area 01F21, 11555 Rockville Pike (first floor), Rockville, Maryland. Publicly available records will be accessible from the Agencywide Documents Access and Management System's (ADAMS) Public Electronic Reading Room on the Internet at the NRC Web site, <http://www.nrc.gov/reading-rm/adams.html>. If you do not have access to ADAMS or if there are problems in accessing the documents located in ADAMS, contact the NRC PDR Reference staff at 1-800-397-4209, 301-415-4737 or by e-mail to pdr@nrc.gov.

Duke Energy Corporation, et al., Docket Nos. 50-413 and 50-414, Catawba Nuclear Station, Units 1 and 2, York County, South Carolina

Date of amendment request: August 19, 2003.

Description of amendment request: The amendments would revise the Technical Specifications (TS) to modify the requirements for the containment pressure control system to eliminate a problem with circuit fluctuation as a result of electronic noise.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the

issue of no significant hazards consideration, which is presented below:

(1) The proposed license amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed amendment has no impact on any accident probabilities or consequences. The CPCS [containment pressure control system] functions to control the operation of the Containment Spray System and the Air Return System following certain design basis accidents. It cannot initiate any accidents by itself. Therefore, accident probabilities will be unaffected. Since the proposed change has been shown to have no effect upon any safety analysis results, the consequences of accidents will also be unaffected.

(2) The proposed license amendment does not create the possibility of a new or different kind of accident from any accident previously evaluated.

As stated previously, the CPCS in and of itself cannot initiate any accident condition. No change to any method of plant operation is being proposed in conjunction with this amendment request. Therefore, no new accident types can be created.

(3) The proposed license amendment does not involve a significant reduction in a margin of safety.

The proposed amendment will have no impact on any safety margin. None of the results of any existing safety analyses is affected as a result of the proposed change. Margin of safety is related to the confidence in the ability of the fission product barriers to perform their design functions. The fission product barriers include the fuel cladding, the reactor coolant pressure boundary, and the containment. None of these fission product barriers will be affected as a result of the proposed change. Therefore, no safety margin will be impacted.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

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NRC Section Chief: John A. Nakoski.

Duke Energy Corporation, Docket Nos. 50-369 and 50-370, McGuire Nuclear Station, Units 1 and 2, and Docket Nos. 50-413 and 50-414, Catawba Nuclear Station, Units 1 and 2, located in Mecklenburg County, North Carolina and York County, South Carolina

Date of amendment request: March 24, 2003, as supplemented June 25, 2003.

Description of amendment request: The proposed amendments would

revise the Technical Specifications (TS) to relocate reactor coolant system cycle-specific parameter limits from the TS to the core operating limits reports for the Catawba and the McGuire Nuclear Stations.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

As required by 10 CFR 50.91(a)(1), this analysis is provided to demonstrate that the proposed license amendment does not involve a significant hazard.

Conformance of the proposed amendment to the standards for a determination of no significant hazards, as defined in 10 CFR 50.92, is shown in the following:

(1) Does the proposed license amendment involve a significant increase in the probability or consequences of an accident previously evaluated?

No. The relocation of Reactor Coolant System (RCS) related cycle-specific parameter limits from the Technical Specifications (TS) to the Core Operating Limits Reports (COLR) proposed by this amendment request does not result in the alteration of the design, material, or construction standards that were applicable prior to the change. The proposed change will not result in the modification of any system interface that would increase the likelihood of an accident since these events are independent of the proposed change. The proposed amendment will not change, degrade, or prevent actions, or alter any assumptions previously made in evaluating the radiological consequences of an accident described in the UFSARS. Therefore, the proposed amendment does not result in the increase in the probability or consequences of an accident previously evaluated.

(2) Does the proposed change create the possibility of a new or different kind of accident from any accident previously evaluated?

No. This change does not create the possibility of a new or different kind of accident from any accident previously evaluated. No new accident causal mechanisms are created as a result of NRC approval of this amendment request. No changes are being made to the facility which should introduce any new accident causal mechanisms. This amendment request does not impact any plant systems that are accident initiators.

(3) Does the proposed change involve a significant reduction in margin of safety?

No. Implementation of this amendment would not involve a significant reduction in the margin of safety. Previously approved methodologies will continue to be used in the determination of cycle-specific core operating limits appearing in the COLRS. Additionally, previously approved RCS minimum total flow rates for McGuire and Catawba are retained in their respective TS