

*Omaha Public Power District, Docket No. 50-285, Fort Calhoun Station, Unit No. 1, Washington County, Nebraska*

*Date of amendment request:* August 28, 2003 (superseded the July 18, 2003, application).

*Description of amendment request:* The proposed amendment will increase the licensed power level to 1524 megawatts thermal (MWt) or 1.60 percent greater than the current power level of 1500 MWt. The requested increase in licensed rated power is the result of a measurement uncertainty recapture (MUR) power uprate. The information provided in support of this request is based on the NRC's Regulatory Issue Summary 2002-03, "Guidance on the Content of Measurement Uncertainty Recapture Power Uprate Applications," dated January 31, 2002.

*Basis for proposed no significant hazards consideration determination:* As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

There are no changes as a result of the MUR power uprate to the design or operation of the plant that could affect system, component, or accident functions. All systems and components function as designed and the performance requirements have been evaluated and found to be acceptable. The reduction in power measurement uncertainty allows for safety analyses to continue to be used without modification. This is because the safety analyses dependent on power level were performed or evaluated at 102% of 1500 MWt (1530 MWt) or higher. Analyses at these power levels support a core power level of 1524 MWt with a measurement uncertainty of 0.4%. Radiological consequences of USAR [Updated Safety Analysis Report] Chapter 14 accidents were assessed previously using the alternate source term methodology (Reference 10.2 [of the August 28, 2003, application]). These analyses were performed at 102% of 1500 MWt (1530 MWt) and continue to be bounding. Updated Safety Analysis Report (USAR) Chapter 14 analyses and accident analyses continue to demonstrate compliance with the relevant accident analyses' acceptance criteria.

Therefore, there is no significant increase in the consequences of any accident previously evaluated.

The primary loop components (reactor vessel, reactor internals, control element drive mechanisms, loop piping and supports, reactor coolant pumps, steam generators, and pressurizer) were evaluated at an uprated core power level of 1524 MWt and continue to comply with their applicable structural limits. These analyses also demonstrate the

components will continue to perform their intended design functions. Changing the heatup and cooldown curves is based on uprated fluence values. This does not have a significant effect on the reactor vessel integrity. Thus, there is no significant increase in the probability of a structural failure of the primary loop components. The LBB [leak-before-break] analysis conclusions remain valid and the breaks previously exempted from structural consideration remain unchanged.

All of the NSSS [nuclear steam system supplier] systems will continue to perform their intended design functions during normal and accident conditions. The auxiliary systems and components continue to comply with the applicable structural limits and will continue to perform their intended functions. The NSSS/BOP [balance-of-plant] interface systems were evaluated at 1524 MWt and will continue to perform their intended design functions. Plant electrical equipment was also evaluated and will continue to perform their intended functions. Therefore, the proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. The proposed change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

No new accident scenarios, failure mechanisms, or single failures are introduced as a result of the proposed change. All systems, structures, and components previously required for the mitigation of an event remain capable of fulfilling their intended design function at the uprated power level. The proposed change has no adverse effects on any safety related systems or component and does not challenge the performance or integrity of any safety related system. Therefore, the proposed change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

3. The proposed change does not involve a significant reduction in a margin of safety.

Operation at 1524 MWt core power does not involve a significant reduction in the margin of safety. The current accident analyses have been previously performed with a 2% power measurement uncertainty or at uprated core powers that exceed the MUR uprated core power. System and component analyses have been completed at the MUR uprated core power conditions. Analyses of the primary fission product barriers at uprated core powers have concluded that all relevant design basis criteria remain satisfied in regard to integrity and compliance with the regulatory acceptance criteria. As appropriate, all evaluations have been both reviewed and approved by the NRC, or are currently under review (the proposed Pressure-Temperature Limits Report). Therefore, the proposed change does not involve a significant reduction in margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are

satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

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*Southern California Edison Company, et al., Docket No. 50-206, San Onofre Nuclear Generating Station, Unit 1, San Diego County, California*

*Date of amendment requests:* July 25, 2003.

*Description of amendment requests:* The amendment application requests a revision to the Unit 1 Defueled Safety Analysis Report (DSAR) that concerns the turbine gantry crane, turbine gantry crane capacity, fuel shipment and the structural descriptions of the turbine building. The licensee is engineering structural changes to the turbine building and gantry crane and replacing the turbine gantry crane trolley in preparation for moving spent fuel from the Unit 1 spent fuel pool to the Independent Spent Fuel Storage Installation (ISFSI). With the planned modifications listed above, the licensee will be able to satisfy the guidance of NUREG-0612, "Control of Heavy Loads at Nuclear Power Plants," and NUREG-0554, "Single-Failure Proof Cranes for Nuclear Power Plants," (regarding safe load handling paths and single-failure proof cranes) in performing the necessary movement of Unit 1 spent fuel to dry cask storage.

*Basis for proposed no significant hazards consideration determination:* As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. Involve a significant increase in the probability or consequences of an accident previously evaluated?

No. The DSAR addresses fuel handling accidents. The process for transporting a cask is essentially unchanged from that previously performed. The building arrangement is such that the cask is never carried over the spent fuel pool. The transport height of the cask has been increased to a minimum of 9 inches based on the design of the new Ederer X-Sam single-failure proof trolley. Because the turbine gantry crane upgrade improves the reliability of the crane, a single failure will not result in loss of its capability to safely retain control of the hook load.

If a portion of the new turbine gantry crane lifting device malfunctions or fails, the crane system is designed such that the load will move a limited distance downward prior to backup restraints becoming engaged. The increased minimum transport height (9 inches) is established to accommodate the