

approved by the NRC, and this proposed change implements that NRC approval. The AST for the Fuel Handling Accident (FHA) takes no credit for containment isolation nor for a filtered release.

(2) Does not create the possibility of a new or different kind of accident from any accident previously evaluated.

The proposed changes to the ITS do not affect nor create a different type of fuel handling accident. The fuel handling accident analyses assume that all of the iodine and noble gases that become airborne, escape, and reach the exclusion area boundary and low population zone with no credit taken for filtration, containment of the source term, or for decay or deposition in the containment. The proposed changes do not involve the addition or modification of equipment nor do they alter the design of plant systems. The revised operations are consistent with the fuel handling accident analyses. Therefore, the proposed changes do not create the possibility of a new or different kind of accident from any accident previously evaluated.

(3) Does not involve a significant reduction in margin of safety.

The calculated doses to both the public and control room operators are well within the limits given in 10 CFR 50.67. The proposed changes do not alter the bases for assurance that safety-related activities are performed correctly or the basis for any ITS that is related to the establishment of or maintenance of a safety margin.

The systems that have been included in the proposed change will have administrative controls in place to assure that the systems are available and can be promptly returned to operation to further reduce dose consequences. These administrative controls will include a single normal or contingency method to promptly close the equipment hatch opening. This prompt method need not completely block the hatch opening nor be capable of resisting pressure, but is to enable the ventilation systems to draw the release from the postulated FHA in the proper direction such that it can be monitored. Therefore, operations of the facility in accordance with the proposed amendment would not involve a significant reduction in margin of safety.

Based on the above, FPC concludes that the proposed license amendment presents no significant hazards consideration under the standards set forth in 10 CFR 50.92(c), and accordingly, a finding of "no significant hazards consideration" is justified.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

Attorney for licensee: R. Alexander Glenn, Associate General Counsel (MAC-BT15A), Florida Power Corporation, P.O. Box 14042, St. Petersburg, Florida 33733-4042.

NRC Section Chief: Allen G. Howe.

*Florida Power Corporation, et al.,
Docket No. 50-302, Crystal River Unit
No. 3 Nuclear Generating Plant, Citrus
County, Florida*

Date of amendment request:
December 19, 2002.

Description of amendment request:
The proposed amendment would revise Crystal River Unit 3 Improved Technical Specification 2.1.1, "Reactor Core Safety Limits." The proposed change will permit the use of the BHTP correlation, which is needed to utilize the Framatome ANP high thermal performance (HTP) spacer grid design.

Basis for proposed no significant hazards consideration determination:
As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

FPC [Florida Power Corporation] has evaluated the proposed License Amendment Request (LAR), which consists of the identified Improved Technical Specification (ITS) change, against the criteria of 10 CFR 50.92(c). The ITS change allows the use of the BHTP Correlation for departure from nucleate boiling (DNB) calculations of reload cores containing the Mark-B/HTP fuel design.

FPC has concluded that this proposed LAR does not involve a significant hazards consideration. The following is a discussion of how each of the criteria is satisfied.

(1) [Does not] [i]nvolve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed safety limit value ensures that fuel integrity will be maintained during normal operations and anticipated operational occurrences (AOOs), and that the design requirements will continue to be met. The proposed methodology for the BHTP departure from nucleate boiling (DNB) correlation will be generically reviewed and approved by the NRC prior to its use by Crystal River Unit 3 (CR-3) in mixed core reload analyses. The core operating limits will be developed in accordance with the new methodology and any limitations established by the NRC in its safety evaluation of the new methodology. The proposed safety limit value does not affect the performance of any equipment used to mitigate the consequences of an analyzed accident. There is no impact on the source term or pathways assumed in accidents previously evaluated. No analysis assumptions are violated and there are no adverse effects on the factors that contribute to offsite or onsite dose as the result of an accident. Therefore, the safety limit value for the BHTP correlation will not involve a significant increase in the probability or consequences of an accident previously evaluated.

(2) [Does not] [c]reate the possibility of a new or different kind of accident from any accident previously evaluated.

The proposed safety limit value does not change the methods governing normal plant operation, nor are the methods utilized to

respond to plant transients altered. The BHTP correlation is not an accident/event initiator. No new initiating events or transients result from the use of the BHTP correlation and the related safety limit changes. Therefore, the safety limit value for the BHTP correlation will not involve the possibility of a new or different kind of accident from any previously evaluated.

(3) [Does not] [i]nvolve a significant reduction in a margin of safety.

The proposed safety limit value has been established in accordance with the methodology for the BHTP correlation, to ensure that the applicable margin of safety is maintained (*i.e.*, there is at least 95% probability at a 95% confidence level that the hot fuel rod in the core does not experience departure from nucleate boiling (DNB)). The proposed methodology for the BHTP DNB correlation will be generically reviewed and approved by the NRC prior to its use by CR-3. The other reactor core safety limits will continue to be met by analyzing the reload for the mixed core using NRC approved methods, and incorporation of resultant operating limits into the Core Operating Limits Report (COLR). Therefore, the safety limit value for the BHTP correlation will not involve a significant reduction in a margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

Attorney for licensee: R. Alexander Glenn, Associate General Counsel (MAC-BT15A), Florida Power Corporation, P.O. Box 14042, St. Petersburg, Florida 33733-4042.

NRC Section Chief: Allen G. Howe.

*Florida Power and Light Company,
Docket Nos. 50-250 and 50-251, Turkey
Point Plant, Units 3 and 4, Miami-Dade
County, Florida*

Date of amendment request:
December 20, 2002.

Description of amendment request:
This proposed amendment provides editorial and administrative changes to the Technical Specifications. The changes correct typographical, spelling, numbering syntax, page break, and font consistency errors as well as removing blank pages and associated references. There are no substantive changes made in the proposed amendment.

Basis for proposed no significant hazards consideration determination:
As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. Will operation of the facility in accordance with this proposed change