

involve a significant increase in the probability or consequences of an accident previously evaluated?

No. The proposed amendments do not involve a significant increase in the probability or consequences of an accident previously evaluated because the proposed amendments are purely administrative or editorial in nature. These amendments make no substantive Technical Specification changes and do not affect any assumptions contained in plant safety analyses, the physical design and/or operation of the plant; and they do not affect Technical Specifications that preserve safety analysis assumptions. Therefore, the proposed changes do not affect the probability or consequences of accidents previously analyzed.

2. Will operation of the facility in accordance with this proposed change create the possibility of a new or different kind of accident from any accident previously evaluated?

No. The use of the administratively changed Technical Specifications does not create the possibility of a new or different kind of accident from any previously evaluated, since the proposed amendments will not change the physical plant or the modes of plant operation defined in the facility operating license. No new failure mode is introduced due to the administrative changes and clarifications, since the proposed changes do not involve the addition or modification of equipment, nor do they alter the design or operation of affected plant systems, structures, or components.

3. Will operation of the facility in accordance with this proposed change involve a significant reduction in a margin of safety?

No. The operating limits and functional capabilities of the affected systems, structures, and components are unchanged by the proposed amendments. The changed Technical Specifications, which correct administrative and editorial errors, and clarify existing Technical Specification requirements, do not reduce any of the margins of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

*Attorney for licensee:* M.S. Ross, Attorney, Florida Power & Light, P.O. Box 14000, Juno Beach, Florida 33408-0420.

*NRC Section Chief:* Allen G. Howe.

*Nebraska Public Power District, Docket No. 50-298, Cooper Nuclear Station, Nemaha County, Nebraska*

*Date of amendment request:*  
December 31, 2002.

*Description of amendment request:*  
The proposed amendment would change the reactor vessel material

surveillance program to incorporate the Boiling Water Reactor Vessel and Internals Project (BWRVIP) Integrated Surveillance Program (ISP) into the licensing basis.

*Basis for proposed no significant hazards consideration determination:*  
As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. Does the proposed change involve a significant increase in the probability or consequences of an accident previously evaluated?

*Response:* No.

Pressure-temperature (P/T) limits (CNS [Cooper Nuclear Station] Technical Specifications Figures 3.4.9-1, 2, and 3) are imposed on the reactor coolant system to ensure that adequate safety margins against non-ductile or brittle fracture exist during normal operation, anticipated operational occurrences, and system hydrostatic tests. The P/T limits are based on the nil-ductility reference temperature,  $RT_{NDT}$ , as described in ASME Section XI, Appendix G. Changes in the fracture toughness properties of RPV [reactor pressure vessel] beltline materials, resulting from the neutron irradiation and the thermal environment, are monitored by a surveillance program in compliance with the requirements of 10 CFR 50 [title 10 of the Code of Federal Regulations part 50] Appendix H. The effect of neutron fluence on the shift in the  $RT_{NDT}$  of RPV materials is predicted by methods given in RG [Regulatory Guide] 1.99, Revision 2.

This change is not related to any accidents previously evaluated. Rather, the reactor vessel surveillance program, corresponding material evaluations, and adjustment of a plant's P/T limits, as necessary, protect against the possibility of reactor vessel brittle fracture. Monitoring, evaluation, and adjustment of CNS P/T limits to ensure adequate margin exists to brittle fracture will continue. This change only replaces a plant-specific monitoring and evaluation program with an integrated industry program, the BWRVIP ISP. The NRC has reviewed this program and approved it for implementation in a Safety Evaluation, dated February 1, 2002.

CNS's current P/T limits were established based on adjusted reference temperatures developed in accordance with the procedures described in RG 1.99, Revision 2. Calculation of adjusted reference temperature by these procedures includes a margin term to ensure conservative, upper-bound values are used for the calculation of the P/T limits. This change does not affect the existing P/T limits in the CNS Technical Specifications Figures 3.4.9-1, 2, and 3. This change will not affect any plant safety limits or limiting conditions of operation. The proposed change will not affect reactor pressure vessel performance as no physical changes are involved aside from changes related to surveillance capsule withdrawal, and CNS vessel P/T limits will remain conservative in accordance with RG 1.99, Revision 2 criteria. The proposed

change will not cause the reactor pressure vessel or interfacing systems to be operated outside of their design or testing limits. Also, the proposed change will not alter any assumptions previously made in evaluating the radiological consequences of accidents. Therefore, the proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. Does the proposed change create the possibility of a new or different kind of accident from any accident previously evaluated?

*Response:* No.

The proposed change revises the CNS license basis to reflect participation in the BWRVIP ISP. Participation in the BWRVIP ISP will continue to ensure that the CNS reactor vessel materials are monitored and evaluated as necessary to protect against brittle fracture. This proposed change does not involve a modification of the design of plant structures, systems, or components. The proposed change will not impact the manner in which the plant is operated as plant operating and testing procedures will not be affected by the change. The proposed change will not degrade the reliability of structures, systems, or components important to safety as equipment protection features will not be deleted or modified, equipment redundancy or independence will not be reduced, supporting system performance will not be downgraded, the frequency of operation of equipment will not be increased, and increased or more severe testing of equipment will not be imposed. No new accident types or failure modes will be introduced as a result of the proposed change. Therefore, the proposed changes do not create the possibility of a new or different kind of accident from that previously evaluated.

3. Does the proposed change involve a significant reduction in a margin of safety?

*Response:* No.

Conformance with 10 CFR [part] 50 Appendix G defines the accepted safety margin for Reactor Coolant Pressure Boundary fracture toughness. The P/T limits are not derived from Design Basis Accident (DBA) analyses. They are prescribed during normal operation to avoid encountering pressure, temperature, and temperature rate of change conditions that might cause undetected flaws to propagate and cause nonductile failure of the reactor pressure vessel, a condition that is unanalyzed. Since the P/T limits are not derived from any DBA, there are no acceptance limits related to the P/T limits. Rather the P/T limits are acceptance limits themselves since they preclude operation in an unanalyzed condition.

This proposed change will not alter the required margins as defined in 10 CFR [part] 50, Appendix G. This proposed change will not affect any safety limits, limiting safety system settings, or limiting conditions of operation. The proposed change does not represent a change in initial conditions, or in a system response time, or in any other parameter affecting the course of an accident analysis supporting the Bases of any Technical Specification. The proposed