

change does not involve revision of the P/T limits. Rather, this change involves a revision to the surveillance capsule withdrawal schedule, a revision to the reactor vessel fluence calculational methodology to achieve consistency within the BWRVIP ISP, and participation in future BWRVIP ISP developments. The current P/T limits were established based on adjusted reference temperatures for vessel beltline materials calculated in accordance with RG 1.99, Revision 2 which will continue to conform to 10 CFR [part] 50 Appendix G. Therefore, the proposed change does not involve a significant reduction in any safety margins.

In summary, it is concluded that this License Amendment Request does not involve significant hazards consideration results. NPPD has researched the existing regulatory precedent and has identified five BWR licensees with similar License Amendment Requests currently under NRC staff review:

- Browns Ferry Units 2 and 3—Submittal date November 6, 2002.
- Monticello Generating Station—Submittal date September 19, 2002.
- River Bend—Submittal date August 15, 2002.
- Fermi Unit 2—Submittal date August 8, 2002.
- Susquehanna Units 1 and 2—Submittal date July 25, 2002.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

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*NRC Section Chief:* Robert A. Gramm.

*Nuclear Management Company, LLC, Docket No. 50-305, Kewaunee Nuclear Power Plant, Kewaunee County, Wisconsin*

*Date of amendment request:* January 13, 2003.

*Description of amendment request:* The proposed amendment would revise the Kewaunee Nuclear Power Plant (KNPP) operating license and Technical Specifications to increase the licensed rated power by 1.4 percent to 1673 megawatts thermal (MWt) using measurement uncertainty recapture.

*Basis for proposed no significant hazards consideration determination:* As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. Operation of the Kewaunee Nuclear Plant in accordance with the proposed amendments does not result in a significant

increase in the probability or consequences of any accident previously evaluated.

There are no changes as a result of the measurement uncertainty recapture (MUR) power uprate to the design or operation of the plant that could affect system, component, or accident mitigative functions. All systems and components will function as designed and the applicable performance requirements have been evaluated and found to be acceptable.

The reduction in power measurement uncertainty allows for some of the safety analyses to continue to be used without modification. This is because the safety analyses were performed or evaluated at either 102 percent of 1650 MWt or higher. Analyses at these power levels support a core power level of 1673 MWt with a measurement uncertainty of 0.6 percent. Radiological consequences of USAR [updated safety analysis report] chapter 14 accidents were assessed previously using the alternate source term (AST) methodology (reference 7.1, TAC [technical assignment control] No. MB4596). These analyses were performed at 102 percent of 1650 MWt and continue to be bounding. The USAR chapter 14 analyses and accident analyses submitted to the NRC [Nuclear Regulatory Commission] with the fuel transition (reference 7.3, TAC No. MB5718) continue to demonstrate compliance with the relevant accident analyses acceptance criteria. Therefore, there is no significant increase in the consequences of any accident previously evaluated.

The primary loop components (reactor vessel, reactor internals, control rod drive mechanisms, loop piping and supports, reactor coolant pumps, steam generators, and pressurizer) were evaluated at an uprated core power level of 1772 MWt and continue to comply with their applicable structural limits. These analyses also demonstrate the components will continue to perform their intended design functions. Changing the applicability of the heatup and cooldown curves is based on uprated fluence values. This does not have a significant effect on the reactor vessel integrity. Thus, there is no significant increase in the probability of a structural failure of the primary loop components.

All of the NSSS [Nuclear Steam Supply System] systems will continue to perform their intended design functions during normal and accident conditions. The auxiliary systems and components continue to comply with the applicable structural limits and will continue to perform their intended functions. The NSSS/BOP [balance of plant] interface systems were evaluated at 1772 MWt and will continue to perform their intended design functions. Plant electrical equipment was also evaluated and will continue to perform their intended functions. Therefore, there is no significant increase in the probability or consequences of an accident previously evaluated.

2. Operation of the Kewaunee Nuclear Power Plant in accordance with the proposed amendments does not result in a new or different kind of accident from any accident previously evaluated.

No new accident scenarios, failure mechanisms, or single failures are introduced

as a result of the proposed change. All systems, structures, and components previously required for the mitigation of an event remain capable of fulfilling their intended design function at the uprated power level. The proposed change has no adverse effects on any safety-related systems or component and does not challenge the performance or integrity of any safety-related system. Therefore, the proposed change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

3. Operation of the Kewaunee Nuclear Power Plant in accordance with the proposed amendments does not result in a significant reduction in a margin of safety.

Operation at the 1673 MWt core power does not involve a significant reduction in the margin of safety. The current accident analyses have been previously performed with a two percent power measurement uncertainty or at uprated core powers that exceed the MUR uprated core power. System and component analyses have been completed at a core power in excess of the MUR uprated core power. Analyses of the primary fission product barriers at uprated core powers have concluded that all relevant design basis criteria remain satisfied in regard to integrity and compliance with the regulatory acceptance criteria. As appropriate, all evaluations have been either reviewed and approved by the NRC, are in the process of being approved by the NRC, or are in compliance with applicable regulatory review guidance and standards. Therefore, the proposed change does not involve a significant reduction in a margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

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*Nuclear Management Company, LLC, Docket Nos. 50-266 and 50-301, Point Beach Nuclear Plant, Units 1 and 2, Town of Two Creeks, Manitowoc County, Wisconsin*

*Date of amendment request:* September 12, 2002.

*Description of amendment request:* The proposed amendment would revise Technical Specification (TS) 3.5.2, "ECCS [Emergency Core Cooling System]—Operating," and TS 3.5.3, "ECCS-Shutdown," to add a surveillance requirement to verify every 31 days that the ECCS piping is full of water; consistent with NUREG-1431, Standard Technical Specifications, Westinghouse Plants, Revision 2.