

either by means of facsimile transmission to 301-415-3725 or by e-mail to OGCMailCenter@nrc.gov. A copy of the request for hearing and petition for leave to intervene should also be sent to the attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for a hearing will not be entertained absent a determination by the Commission, the presiding officer or the Atomic Safety and Licensing Board that the petition and/or request should be granted based upon a balancing of factors specified in 10 CFR 2.714(a)(1)(i)-(v) and 2.714(d).

For further details with respect to this action, see the application for amendment which is available for public inspection at the Commission's PDR, located at One White Flint North, Public File Area 01F21, 11555 Rockville Pike (first floor), Rockville, Maryland. Publicly available records will be accessible from the Agencywide Documents Access and Management System's (ADAMS) Public Electronic Reading Room on the Internet at the NRC Web site, <http://www.nrc.gov/reading-rm/adams.html>. If you do not have access to ADAMS or if there are problems in accessing the documents located in ADAMS, contact the NRC PDR Reference staff at 1-800-397-4209, 301-415-4737 or by e-mail to pdr@nrc.gov.

**Exelon Generation Company, LLC,
Docket Nos. 50-373 and 50-374,
LaSalle County Station, Units 1 and 2,
LaSalle County, Illinois**

Date of amendment request: July 1, 2003.

Description of amendment request: The proposed amendments would revise Appendix A, Technical Specifications (TS), of Facility Operating License Nos. NPF-11 and NPF-18. Specifically, the proposed changes will delete one and add two references to the list of analytical methods in TS 5.6.5, "Core Operating Limits Report (COLR)," that can be used to determine core operating limits. The deleted reference is to an analytical method that is no longer applicable to LaSalle County Station (LSCS). The new references will allow LSCS to use General Electric Company (GE) methods for the determination of fuel assembly critical power of Framatome Advanced Nuclear Fuel, Inc. (Framatome) Atrium-9B and Atrium-10 fuel. The proposed changes are the result of a LSCS decision to insert GE14 fuel during the upcoming refueling outage at LSCS Unit 1 in January 2004. GE's safety analysis methodologies have been previously

used at LSCS and GE14 fuel is currently in use at other Exelon Generation Company, LLC (Exelon), stations.

The first added reference, "GEXL96 Correlation for Atrium-9B Fuel," will list a method that was previously approved by the NRC for use by licensees. The second added reference, "GEXL97 Correlation for Atrium-10 Fuel," will list a GE method for determining the critical power for Atrium-10 fuel. This correlation has not been previously reviewed and approved by the NRC for use by licensees. Additionally, editorial changes will be made to existing references.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The proposed changes do not involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed changes will delete one and add two additional references to the list of administratively controlled analytical methods in TS 5.6.5, "Core Operating Limits Report (COLR)," that can be used to determine core operating limits and make minor editorial changes to the existing references. TS 5.6.5 lists NRC approved analytical methods used at LaSalle County Station (LSCS) to determine core operating limits. [LSCS Unit 1 is scheduled to load GE fuel during its upcoming outage in January 2004.]

The proposed changes to TS Section 5.6.5 will add the fuel analytical methods that support the initial insertion of GE14 fuel to the list of methods used to determine the core operating limits. The deletion or addition of approved methods to TS Section 5.6.5 and minor editorial changes to the existing references has no effect on any accident initiator or precursor previously evaluated and does not change the manner in which the core is operated. The methods have been reviewed to ensure that the output accurately models predicted core behavior, have no effect on the type or amount of radiation released, and have no effect on predicted offsite doses in the event of an accident. Thus, the proposed changes do not have any effect on the probability of an accident previously evaluated.

The proposed changes in the administratively controlled analytical methods does [do] not affect the ability of LSCS to successfully respond to previously evaluated accidents and does [do] not affect radiological assumptions used in the evaluations. Thus, the radiological consequences of any accident previously evaluated are not increased.

Therefore, the proposed changes do not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. The proposed changes do not create the possibility of a new or different kind of

accident from any accident previously evaluated.

The proposed changes to TS Section 5.6.5 do not affect the performance of any LSCS structure, system, or component credited with mitigating any accident previously evaluated. The insertion of a new generation of fuel which has been analyzed with NRC approved methodologies will not affect the control parameters governing unit operation or the response of plant equipment to transient conditions. The proposed changes do not introduce any new modes of system operation or failure mechanisms.

Therefore, the proposed changes do not create the possibility of a new or different kind of accident from any previously evaluated.

3. The proposed changes do not involve a significant reduction in a margin of safety.

The proposed changes will delete one and add two additional references to the list of administratively controlled analytical methods in TS 5.6.5 that can be used to determine core operating limits and make minor editorial changes to the titles of existing references. The proposed changes do not modify the safety limits or setpoints at which protective actions are initiated, and do not change the requirements governing operation or availability of safety equipment assumed to operate to preserve the margin of safety. Therefore, LSCS has determined that the proposed changes provide an equivalent level of protection as that currently provided.

Therefore, the proposed changes do not involve a significant reduction in a margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the requested amendments involve no significant hazards consideration.

Attorney for licensee: Mr. Edward J. Cullen, Deputy General Counsel, Exelon BSC—Legal, 2301 Market Street, Philadelphia, PA 19101.

NRC Section Chief: Anthony J. Mendiola.

**Maine Yankee Atomic Power Company,
Docket No. 50-309, Maine Yankee
Atomic Power Station, Lincoln County,
Maine**

Date of amendment request: September 11, 2003.

Description of amendment request: Revise the dose model for the containment activated concrete, rebar (hereafter referred to as activated concrete) and liner, by incorporating more realistic radionuclide release rates and to change the associated derived concentration guideline limit (DCGL) for activated concrete.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the