

deletes an erroneous approximate value from the description of test conditions for maximum allowable leakage from recirculation heat removal system components, is consistent with the existing plant design as described in the Palisades Final Safety Analysis Report. The proposed correction to TS 5.6.1 is editorial in nature and is consistent with the Nuclear Regulatory Commission approved standard technical specifications. The proposed amendment does not involve operation of the required structures, systems or components (SSCs) in a manner or configuration different from those previously recognized or evaluated.

Therefore, operation of the facility in accordance with the proposed amendment would not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. Create the possibility of a new or different kind of accident from any accident previously evaluated.

The proposed amendment does not involve a physical alteration of any SSC or a change in the way any SSC is operated. The proposed amendment does not involve operation of any required SSCs in a manner or configuration different from those previously recognized or evaluated. No new failure mechanisms will be introduced by the changes being requested.

Thus, this change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

3. Involve a significant reduction in a margin of safety.

The proposed amendment does not affect any margin of safety. The proposed amendment does not involve any physical changes to the plant or manner in which the plant is operated.

Therefore, the proposed amendment would not involve a significant reduction in a margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

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NRC Section Chief: L. Raghavan.

Southern Nuclear Operating Company, Inc., Docket Nos. 50-348, Joseph M. Farley Nuclear Plant, Unit 1, Houston County, Alabama

Date of amendment request: September 19, 2003.

Description of amendment request: The proposed amendment would revise Technical Specifications (TS) Limiting Conditions for Operation (LCO) 3.8.4, "DC Sources—Operating," for the remainder of operating cycle 19.

Specifically, the proposed TS change would increase the Completion Time for the 1B Auxiliary Building DC electrical power system inoperability due to an inoperable battery to allow for on-line replacement of individual cells. Cycle 19 is presently scheduled to end on October 2, 2004.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The proposed changes do not involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed change to LCO 3.8.4 creates an extended Completion Time for an inoperable 1B Auxiliary Building DC electrical power subsystem due to an inoperable battery on Unit 1 only for the remainder of operating cycle 19. The Auxiliary Building battery is not a direct initiator of any analyzed accident sequence. The radiological consequences of any associated accidents are not impacted by the proposed amendment. Therefore, the proposed amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. The proposed changes do not create the possibility of a new or different kind of accident from any accident previously evaluated.

The proposed change involves no change to the physical plant. It allows additional time for corrective maintenance on the 1B Auxiliary Building battery on Unit 1. The proposed amendment involves an extension of a previously determined acceptable mode of operation. The proposed amendment does not introduce any new equipment, create new failure modes for existing equipment, or create any new limiting single failures. Therefore, the proposed amendment does not create the possibility of a new or different kind of accident from any accident previously evaluated.

3. The proposed changes do not involve a significant reduction in a margin of safety.

The physical plant is unaffected by these changes. The proposed changes do not impact accident offsite dose, containment pressure or temperature, emergency core cooling system (ECCS) or reactor protection system (RPS) settings or any other parameter that could affect a margin of safety. Under the proposed amendment, the unit will continue to be operated in a condition that will ensure that emergency power will be available as needed. The extended Completion Time for an inoperable battery has been shown to have a very small impact on plant risk using the criteria of Regulatory Guides 1.174, An Approach for Using Probabilistic Risk Assessments in Risk-Informed Decision-making and 1.177, An Approach for Plant-Specific Risk-Informed Decisionmaking: Technical Specifications and is acceptable. Therefore, the proposed amendment does not

involve a significant reduction in a margin to safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

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NRC Section Chief: John A. Nakoski.

Southern Nuclear Operating Company, Inc., Docket Nos. 50-348 and 50-364, Joseph M. Farley Nuclear Plant, Units 1 and 2, Houston County, Alabama

Date of amendment request: August 29, 2003.

Description of amendment request: The proposed amendments would revise Technical Specifications Limiting Condition of Operation 3.9.3, "Containment Penetrations." The proposed changes would allow the equipment hatch to be open during core alterations and/or during movement of irradiated fuel assemblies within containment.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. Do the proposed changes involve a significant increase in the probability or consequences of an accident previously evaluated?

No. The proposed changes will allow the equipment hatch to be open during core alterations and movement of irradiated fuel assemblies inside containment. The proposed changes will not alter the manner in which fuel is handled or core alterations are performed. The equipment hatch is not an initiator of any accident. The status of the equipment hatch during refueling operations has no effect on the probability of the occurrence of any accident previously evaluated. The radiological consequences of a fuel handling accident inside containment have been determined to be well within the limits of 10 CFR 100 and they meet the acceptance criteria of General Design Criterion (GDC) 19. Therefore the proposed changes do not involve a significant increase in the probability or consequences of [any] accident previously evaluated.

2. Do the proposed changes create the possibility of a new or different kind of accident from any previously evaluated?

No. The proposed changes do not create any new failure modes for any system or component, nor do they adversely affect plant operation. No new equipment will be added and no new limiting single failures