

ADAMS, should contact the NRC PDR Reference staff by telephone at 1-800-397-4209, 301-415-4737, or by e-mail to pdr@nrc.gov.

Dated at Rockville, Maryland, this 6th day of November, 2003.

For the Nuclear Regulatory Commission.

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NUCLEAR REGULATORY COMMISSION

[Docket No. 50-425]

Southern Nuclear Operating Company, Inc.; Notice of Consideration of Issuance of Amendment to Facility Operating License, Proposed No Significant Hazards Consideration Determination, and Opportunity for a Hearing

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License No. NPF-81, issued to Southern Nuclear Operating Company, *et al.* (SNC, the licensee), for operation of the Vogtle Electric Generating Plant, Unit 2, located in Burke County, Georgia.

The proposed amendment would extend the surveillance interval for the Memories Test portion of the ACTUATION LOGIC TEST for: (1) Power Range Block (Switch position 1), (2) Intermediate Range Block (Switch position 2), (3) Source Range Block (Switch positions 3 and 4), (3) Safety Injection (SI) Block, Pressurizer (Switch positions 5 and 6), (4) SI Block, High Steam Pressure Rate (Switch positions 7 and 8), (5) Auto SI Block (Switch position 9), and (6) Feedwater Isolation on P14 or SI (Switch positions 10 and 11). In addition to the functions listed above, the licensee is requesting an extension of the surveillance interval for the portions of the ACTUATION LOGIC TEST for Feedwater Isolation on P14 or SI that pass through the memories circuits and the Power Range block of the Source Range Trip test for the Unit 2 Train B Solid State Protection System to the next refueling outage at the end of Cycle 10 or the next Unit 2 shutdown to MODE 5, whichever comes first.

Because the above-described surveillances will become due multiple times before the end of the current fuel cycle, and the Memories Test Switch is not functioning, the licensee is

requesting an exigent Technical Specification change in accordance with 10 CFR 50.91(a)(6) to extend the surveillance interval of the above-described tests. SNC is requesting that the surveillance interval be extended to the end of the current cycle (Cycle 10) or the next Unit 2 shutdown to MODE 5, whichever comes first.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

Pursuant to 10 CFR 50.91(a)(6) for amendments to be granted under exigent circumstances, the NRC staff must determine that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. Does the proposed change involve a significant increase in the probability or consequences of an accident previously evaluated?

The proposed change does not physically alter any plant structures, systems or components. The SSPS [Solid State Protection System] at VEGP [Votgle Electric Generating Plant] has a history of high reliability. In addition, similar changes to the surveillance interval for actuation logic testing for Westinghouse SSPS actuation logic has been approved by the NRC with their approval of WCAP-15376 and Technical Specification Task Force (TSTF) 411. Therefore[,] there will not be a significant increase in the probability of an accident previously evaluated. There will not be a significant increase in the consequences of any accident previously evaluated as a result of this Technical Specification amendment because the incremental condition large early release probability is very small in accordance with the criteria of Regulatory Guide 1.177. Therefore, the proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. Does the proposed change create the possibility of a new or different kind of accident from any accident previously evaluated?

The proposed change involves an extension of a previously determined acceptable surveillance interval. The

proposed change does not introduce any new equipment, create new failure modes for existing equipment, or create any new limiting single failures. In addition, compensatory actions will be in place which will offset the very small increase in risk. Therefore, the requested Technical Specification amendment does not create the possibility of a new or different kind of accident from any accident previously evaluated.

3. Does the proposed change involve a significant reduction in a margin of safety.

The extended surveillance interval for the SSPS ACTUATION LOGIC TEST has been shown to have a very small impact on plant risk using the criteria of Regulatory Guides 1.174 and 1.177. In addition, compensatory actions in place will be in place in the case of a failure of the functions listed above. Therefore, the enforcement discretion does not involve a significant reduction in a margin to safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

The Commission is seeking public comments on this proposed determination. Any comments received within 14 days after the date of publication of this notice will be considered in making any final determination.

Normally, the Commission will not issue the amendment until the expiration of the 14-day notice period. However, should circumstances change during the notice period, such that failure to act in a timely way would result, for example, in derating or shutdown of the facility, the Commission may issue the license amendment before the expiration of the 14-day notice period, provided that its final determination is that the amendment involves no significant hazards consideration. The final determination will consider all public and State comments received. Should the Commission take this action, it will publish in the **Federal Register** a notice of issuance. The Commission expects that the need to take this action will occur very infrequently.

Written comments may be submitted by mail to the Chief, Rules and Directives Branch, Division of Administrative Services, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and should cite the publication date and page number of this **Federal Register** notice. Written comments may also be delivered to Room 6D59, Two White Flint North, 11545 Rockville Pike, Rockville, Maryland, from 7:30