

The proposed changes would increase the allowed outage time for inoperable Remote Shutdown System components to a time that is more consistent with their safety significance. It would also relocate the description of the required components to the Bases where it will be directly controlled by the licensee.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. Does the proposed change involve a significant increase in the probability or consequences of an accident previously evaluated?

Response: No.

Because the proposed changes do not involve potential accident initiators, there is no significant increase in the probability of an accident previously evaluated. There is no proposed change to the design basis or configuration of the plant and the extension of the allowed outage time of the Remote Shutdown System functions does not have a significant effect on safety. Consequently there is no significant increase in the consequences of an accident previously evaluated.

2. Does the proposed change create the possibility of a new or different kind of accident from any accident previously evaluated?

Response: No.

The proposed changes do not affect how the plant is operated or involve any physical changes to the plant. Therefore there is no possibility of a new or different kind of accident from any previously evaluated.

3. Does the proposed change involve a significant reduction in a margin of safety?

Response: No.

Except for extending the allowed outage time for Remote Shutdown System function from 7 days to 30 days, the proposed changes are essentially administrative. The evaluation of the extension of the allowed outage time demonstrated that there was no significant reduction in the margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the request for amendments involves no significant hazards consideration.

Attorney for licensee: A. H. Gutterman, Esq., Morgan, Lewis & Bockius, 1111 Pennsylvania Avenue, NW., Washington, DC 20004.

NRC Section Chief: Robert A. Gramm.

Notice of Issuance of Amendments to Facility Operating Licenses

During the period since publication of the last biweekly notice, the Commission has issued the following amendments. The Commission has determined for each of these

amendments that the application complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment.

Notice of Consideration of Issuance of Amendment to Facility Operating License, Proposed No Significant Hazards Consideration Determination, and Opportunity for A Hearing in connection with these actions was published in the **Federal Register** as indicated.

Unless otherwise indicated, the Commission has determined that these amendments satisfy the criteria for categorical exclusion in accordance with 10 CFR 51.22. Therefore, pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared for these amendments. If the Commission has prepared an environmental assessment under the special circumstances provision in 10 CFR 51.12(b) and has made a determination based on that assessment, it is so indicated.

For further details with respect to the action see (1) the applications for amendment, (2) the amendment, and (3) the Commission's related letter, Safety Evaluation and/or Environmental Assessment as indicated. All of these items are available for public inspection at the Commission's Public Document Room, located at One White Flint North, Public File Area 01F21, 11555 Rockville Pike (first floor), Rockville, Maryland. Publicly available records will be accessible from the Agencywide Documents Access and Management Systems (ADAMS) Public Electronic Reading Room on the Internet at the NRC Web site, <http://www.nrc.gov/reading-rm/adams.html>. If you do not have access to ADAMS or if there are problems in accessing the documents located in ADAMS, contact the NRC Public Document Room (PDR) Reference staff at 1-800-397-4209, 301-415-4737 or by e-mail to pdr@nrc.gov.

Detroit Edison Company, Docket No. 50-341, Fermi 2, Monroe County, Michigan.

Date of application for amendment: June 24, 2003.

Brief description of amendment: The amendment revises Technical Specification 3.1.8, "Scram Discharge Volume (SDV) Vent and Drain Valves," to allow a vent or drain line with one inoperable valve to be isolated instead of requiring the valve to be restored to Operable status within 7 days.

Date of issuance: October 30, 2003.

Effective date: As of the date of issuance and shall be implemented within 90 days.

Amendment No.: 157.

Facility Operating License No. NPF-43: Amendment revises the Technical Specifications.

Date of initial notice in Federal Register: August 19, 2003 (68 FR 49815).

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated October 30, 2003.

No significant hazards consideration comments received: No.

Duke Energy Corporation, Docket Nos. 50-269, 50-270, and 50-287, Oconee Nuclear Station, Units 1, 2, and 3, Oconee County, South Carolina.

Date of application of amendments: July 10, 2003.

Brief description of amendments: The amendments revised the Technical Specifications to remove requirements that are no longer applicable because the implementation of the automatic feedwater isolation system modification has been completed on all three Oconee units.

Date of Issuance: November 5, 2003.

Effective date: As of the date of issuance and shall be implemented within 30 days from the date of issuance.

Amendment Nos.: 336, 336, & 337.

Renewed Facility Operating License Nos. DPR-38, DPR-47, and DPR-55: Amendments revised the Technical Specifications.

Date of initial notice in Federal Register: August 19, 2003 (68 FR 49816).

The Commission's related evaluation of the amendments is contained in a Safety Evaluation dated November 5, 2003.

No significant hazards consideration comments received: No.

Entergy Operations, Inc., System Energy Resources, Inc., South Mississippi Electric Power Association, and Entergy Mississippi, Inc., Docket No. 50-416, Grand Gulf Nuclear Station, Unit 1, Claiborne County, Mississippi.

Date of application for amendment: April 3, 2003.

Brief description of amendment: The changes revise the Updated Final Safety Analysis Report to change the Reactor Vessel Material Surveillance Program. The change reflects participation in the Boiling Water Reactor Vessel and Internals Project Integrated Surveillance Program.

Date of issuance: November 4, 2003.

Effective date: As of the date of issuance and shall be implemented within 60 days of issuance.