

*Amendment No.* 160.

*Facility Operating License No. NPF-29:* The amendment revises the Updated Final Safety Analysis Report.

*Date of initial notice in **Federal Register**:* May 13, 2003 (68 FR 25653).

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated November 4, 2003.

No significant hazards consideration comments received: No.

*Entergy Nuclear Operations, Inc., Docket No. 50-286, Indian Point Nuclear Generating Unit No. 3, Westchester County, New York.*

*Date of application for amendment:* October 23, 2001, as supplemented on March 29 and December 17, 2002, and June 12, 2003.

*Brief description of amendment:* The amendment revised Technical Specification (TS) 5.5.10, "Ventilation Filter Testing Program," to adopt the requirements of the American Society for Testing and Materials Standard D3803-1989, "Standard Test Method for Nuclear-Grade Activated Carbon." The TS revisions are in response to Nuclear Regulatory Commission (NRC) Generic Letter (GL) 99-02, "Laboratory Testing of Nuclear-Grade Activated Charcoal." The amendment revises the TSs: (1) To provide a control room ventilation system (CRVS) methyl iodide removal efficiency of greater than or equal to 95.5% and remove the notation that there is a 1-inch charcoal bed depth; (2) to allow for the continued use of the existing CRVS through Refueling Outage 13, in order to design, fabricate, and install a 2-inch charcoal filter bed; and (3) to add a note in the TS requiring a demonstration of charcoal efficiency of 93% when changing the charcoal in the existing CRVS bed prior to any fuel movement in the upcoming Refueling Outage 12 and every 6 months thereafter until the new beds are installed. The NRC had previously published a notice of consideration on December 12, 2001 (66 FR 64292) regarding a similar proposal from the licensee in response to GL 99-02. However, in response to a request for additional information from the NRC dated March 29, 2002, the licensee revised its application and withdrew the prior request to change the maximum CRVS differential pressure in TS 5.5.10.d.

*Date of issuance:* October 30, 2003.

*Effective date:* As of the date of issuance and shall be implemented 30 days from the date of issuance.

*Amendment No.:* 219.

*Facility Operating License No. DPR-64:* Amendment revised the Technical Specifications.

*Date of initial notice in **Federal Register**:* March 18, 2003 (68 FR 12951).

The March 29 and December 17, 2002, and June 12, 2003, letters provided clarifying information that did not enlarge the scope of the amendment request or change the initial proposed no significant hazards consideration determination.

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated October 30, 2003.

No significant hazards consideration comments received: No.

*Entergy Operations, Inc., Docket No. 50-382, Waterford Steam Electric Station, Unit 3, St. Charles Parish, Louisiana.*

*Date of amendment request:* December 16, 2002, as supplemented by letters dated July 30, and September 29, 2003.

*Brief description of amendment:* The amendment adds Combustion Engineering topical report CEN-372-P-A, May 1990, "Fuel Rod Maximum Allowable Gas Pressure," to the list of topical reports in Technical Specification 6.9.1.11.1, used to determine the Waterford Steam Electric Station, Unit 3 core operating limits. In addition, the amendment approves the deletion of applicable dates and revision numbers for CEN-372-P-A and other topical reports listed in TS 6.9.1.11.1.

*Date of issuance:* October 31, 2003.

*Effective date:* As of the date of issuance and shall be implemented 60 days from the date of issuance.

*Amendment No.:* 191.

*Facility Operating License No. NPF-38:* The amendment revised the Technical Specifications.

*Date of initial notice in **Federal Register**:* February 4, 2003 (68 FR 5673). The July 30, and September 29, 2003, supplemental letters provided clarifying information that did not change the scope of the original **Federal Register** notice or the original no significant hazards consideration determination.

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated October 31, 2003.

No significant hazards consideration comments received: No.

*Exelon Generation Company, LLC, Docket Nos. 50-373 and 50-374, LaSalle County Station, Units 1 and 2, LaSalle County, Illinois.*

*Date of application for amendments:* March 31, 2003.

*Brief description of amendments:* The amendments revise Appendix A,

Technical Specifications (TS), of Facility Operating License Nos. NPF-11 and NPF-18. Specifically, the changes modify TS 5.7, "High Radiation Area," by incorporating the wording and requirements from NUREG-1434, "Standard Technical Specifications General Electric Plants, BWR/6," Revision 2, dated June 2001. The revision also includes administrative changes regarding access control and terminology for high radiation areas.

*Date of issuance:* October 31, 2003.

*Effective date:* As of the date of issuance and shall be implemented within 60 days.

*Amendment Nos.:* 161/147.

*Facility Operating License Nos. NPF-11 and NPF-18:* The amendments revised the Technical Specifications.

*Date of initial notice in **Federal Register**:* May 27, 2003 (68 FR 28852).

The Commission's related evaluation of the amendments is contained in a Safety Evaluation dated October 31, 2003.

No significant hazards consideration comments received: No.

*Exelon Generation Company, LLC, Docket Nos. 50-352 and 50-353.*

*Limerick Generating Station, Units 1 and 2, Montgomery County, Pennsylvania.*

*Date of application for amendments:* December 20, 2002, as supplemented May 30, 2003.

*Brief description of amendments:* The amendments removed the current facility reactor material specimen surveillance schedule from the Technical Specifications for Limerick Generating Station, Units 1 and 2 (LGS-1 and 2). The licensee also revised the Updated Final Safety Analysis Report (UFSAR) for LGS-1 and 2 to reflect implementation of the Boiling Water Reactor Vessel and Internals Project reactor pressure vessel integrated surveillance program as the basis for demonstrating the compliance with the requirements of Appendix H, "Reactor Vessel Material Surveillance Program Requirements," to title 10 of the Code of Federal Regulations, Part 50.

*Date of issuance:* November 4, 2003.

*Effective date:* As of the date of issuance and shall be implemented within 30 days.

*Amendment Nos.:* 167 and 130.

*Facility Operating License Nos. NPF-39 and NPF-85:* The amendments revised the Technical Specifications and authorized changes to the UFSAR for LGS-1 and 2.

*Date of initial notice in **Federal Register**:* February 4, 2003 (68 FR 5669). The supplement dated May 30, 2003, provided additional information that