

withdrawn or fully inserted while maintaining the insertion limits of LCO 3.1.6, LCO 3.1.7 and LCO 3.1.8." The TS Bases make no restriction or condition limiting only one CEA within a subgroup, to having only one CEA position indication channel OPERABLE. Therefore, modifying the wording for LCO 3.1.5, Condition B, does not involve a significant reduction in the margin of safety since loss of indication to more than one CEA is already considered in the licensing bases.

Therefore, the proposed changes do not involve a significant reduction in the margin of safety.

Based on the above, APS [Arizona Public Service] concludes that the activities associated with the proposed amendment(s) present no significant hazards consideration under the standards set forth in 10 CFR 50.92 ["Issuance of Amendment,"] (c) and, accordingly, a finding of "no significant hazards consideration" is justified.

The NRC staff has reviewed the licensee's analysis and, based on that review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the request for amendments involves no significant hazards consideration.

Attorney for licensee: Kenneth C. Manne, Senior Attorney, Arizona Public Service Company, P.O. Box 52034, Mail Station 7636, Phoenix, Arizona 85072-2034.

NRC Section Chief: Stephen Dembek.

Arizona Public Service Company, et al.
Docket Nos. STN 50-528, STN 50-529, and STN 50-530, Palo Verde Nuclear Generating Station, Units 1, 2, and 3, Maricopa County, Arizona

Date of amendment request: October 7, 2003.

Description of amendment request: The licensee is proposing to revise Technical Specification (TS) Section 5.5.6, "Pre-Stressed Concrete Containment Tendon Surveillance Program," for consistency with the requirements of 10 CFR 50.55a(g)(4) for components classified as Code Class CC. The proposed revision to TS 5.5.6 is to indicate that the Containment Tendon Surveillance Program, inspection frequencies, and acceptance criteria shall be in accordance with Section XI, Subsection IWL of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code and the applicable addenda as required by 10 CFR 50.55a, except where an exemption or relief has been authorized by the NRC. The licensee has also proposed to delete the provisions of Surveillance Requirement 3.0.2 from this specification.

In addition, the licensee is proposing to revise TS 5.5.16, "Containment Leakage Rate Testing Program," to add exceptions to Regulatory Guide 1.163, "Performance-Based Containment Leak-Test Program."

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed changes would revise Technical Specification (TS) Section 5.5.6, "Pre-Stressed Concrete Containment Tendon Surveillance Program," and Section 5.5.16, "Containment Leakage Rate Testing Program," for consistency with the requirements of 10 CFR 50.55a(g)(4) for components classified as Code Class CC. The revised requirements do not affect the function of the containment post-tensioning system components. The post-tensioning systems are passive components whose failure modes could not act as accident initiators or precursors. The improved inspections required by the American Society of Mechanical Engineers (ASME) Code serve to maintain containment response to accident conditions, by causing the identification and repair of defects in the containment.

The proposed change affects the frequency of visual examinations that will be performed for the concrete surfaces of the containment for the purpose of the Containment Leakage Rate Testing Program. In addition, the proposed change allows those examinations to be performed during power operation as opposed to during a refueling outage. The frequency of visual examinations of the concrete surfaces of the containment and the mode of operation during which those examinations are performed has no relationship to or adverse impact on the probability of any of the initiating events assumed in the accident analyses. The proposed change would allow visual examinations that are performed pursuant to NRC approved ASME Code Section XI requirements (except where relief has been granted by the NRC) to meet the intent of visual examinations [as] required by Regulatory Guide 1.163, "Performance-Based Containment Leak-Test Programs," without requiring additional visual examinations pursuant to the Regulatory Guide. The intent of early detection of deterioration will continue to be met by the more rigorous requirements of the ASME Code [required] visual examinations. As such, the safety function of the containment as a fission product barrier is maintained.

The proposed amendment does not impact any accident initiators, analyzed events, or assumed mitigation of accident or transient events. The proposed changes do not involve the addition or removal of any equipment or any design changes to the facility.

Therefore, this proposed change does not represent a significant increase in the probability or consequences of an accident previously evaluated.

2. The proposed change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

The proposed change revises the Technical Specification administrative controls programs for consistency with the requirements of 10 CFR 50.55a(g)(4) for components classified as Code Class CC. The function of the containment post-tensioning system components are not altered by this change. The improved inspections required

by the American Society of Mechanical Engineers (ASME) Code serve to maintain containment response to accident conditions, by causing the identification and repair of defects in the containment. In addition, the change affects the frequency of visual examinations that will be performed for the concrete surface containments. The proposed change also allows those examinations to be performed during power operation as opposed to during a refueling outage. Therefore, this change updates the Technical Specifications to meet the current regulations and eliminates duplication of requirements. The safety function of the containment as a fission product barrier will be maintained.

Therefore, this proposed change does not create the possibility of an accident of a different kind than previously evaluated.

3. The proposed change does not involve a significant reduction in a margin of safety.

The proposed change revises the improved Standard Technical Specification administrative controls programs for consistency with the requirements of 10 CFR 50.55a(g)(4) for components classified as Code Class CC. The function of the containment post-tensioning system components are not altered by this change. The change also affects the frequency of visual examinations that will be performed for the concrete surface containments. In addition, the proposed change allows those examinations to be performed during power operation as opposed to during a refueling outage. The change ensures that containment integrity [will be maintained] and ensures that the safety function of the containment as a fission product barrier will be maintained.

Therefore, this proposed change does not involve a significant reduction in a margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendments request involves no significant hazards consideration.

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NRC Section Chief: Stephen Dembek.

Dominion Nuclear Connecticut, Inc.,
Docket No. 50-245, Millstone Power Station, Unit No. 1, New London County, Connecticut

Date of amendment request: September 18, 2003.

Description of amendment request: The licensee is proposing to revise the Design Features Technical Specification 4.2, "Fuel Storage." The licensee's technical specification change implements the following proposed changes:

(1) Eliminates all credit for Boraflex as a neutron absorber.