

Therefore, the proposed changes do not create the possibility of a new or different kind of accident from those previously analyzed.

3. Involve a significant reduction in the margin of safety.

The implementation of the proposed changes does not reduce the margin of safety. The proposed changes for the MCR/ESGR EVS, PREACS, and MCE [MCR]/ESGR Bottled Air System do not affect the ability of these systems to perform their intended safety functions to maintain dose less than the required limits during design basis radiological events. The revised dose calculations also indicate that the change to the containment depressurization times will continue to maintain the dose to the public and control room operators less than the required limits.

The radiological analysis results, when compared with the revised TEDE [total effective dose equivalent] acceptance criteria, meet the applicable limits. These acceptance criteria have been developed for application to analyses performed with alternative source terms. These acceptance criteria have been developed for the purpose of use in design basis accident analyses such that meeting the stated limits demonstrates adequate protection of public health and safety. It is thus concluded that the margin of safety will not be reduced by the implementation of the changes.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

Attorney for licensee: Ms. Lillian M. Cuoco, Esq., Senior Counsel, Dominion Resources Services, Inc., Millstone Power Station, Building 475, 5th Floor, Rope Ferry Road, Rt. 156, Waterford, Connecticut 06385.

NRC Section Chief: John A. Nakoski.

Notice of Issuance of Amendments to Facility Operating Licenses

During the period since publication of the last biweekly notice, the Commission has issued the following amendments. The Commission has determined for each of these amendments that the application complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment.

Notice of Consideration of Issuance of Amendment to Facility Operating License, Proposed No Significant Hazards Consideration Determination, and Opportunity for a Hearing in

connection with these actions was published in the **Federal Register** as indicated.

Unless otherwise indicated, the Commission has determined that these amendments satisfy the criteria for categorical exclusion in accordance with 10 CFR 51.22. Therefore, pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared for these amendments. If the Commission has prepared an environmental assessment under the special circumstances provision in 10 CFR 51.12(b) and has made a determination based on that assessment, it is so indicated.

For further details with respect to the action see (1) The applications for amendment, (2) the amendment, and (3) the Commission's related letter, Safety Evaluation and/or Environmental Assessment as indicated. All of these items are available for public inspection at the Commission's Public Document Room, located at One White Flint North, Public File Area 01F21, 11555 Rockville Pike (first floor), Rockville, Maryland. Publicly available records will be accessible from the Agencywide Documents Access and Management Systems (ADAMS) Public Electronic Reading Room on the Internet at the NRC Web site, <http://www.nrc.gov/reading-rm/adams.html>. If you do not have access to ADAMS or if there are problems in accessing the documents located in ADAMS, contact the NRC Public Document Room (PDR) Reference staff at 1-800-397-4209, 301-415-4737 or by e-mail to pdr@nrc.gov.

AmerGen Energy Company, LLC, et al., Docket No. 50-219, Oyster Creek Nuclear Generating Station, Ocean County, New Jersey

Date of application for amendment: June 2, 2003.

Brief description of amendment: The amendment revised the Technical Specifications, Sections 3.7.B.1 and 3.7.C.2. Section 3.7.B.1 required that the reactor may remain in operation "for a period not to exceed 7 days in any 30 day period if a startup transformer is out of service." Section 3.7.C.2 required that the reactor may be in operation "for a period not to exceed 7 days in any 30 day period if a diesel generator is out of service." The amendment deleted the phrase "in any 30 day period" from these two sections.

Date of Issuance: November 24, 2003.

Effective date: November 24, 2003 and shall be implemented within 30 days of issuance.

Amendment No.: 239.

Facility Operating License No. DPR-16: Amendment revised the Technical Specifications.

Date of initial notice in Federal Register: July 8, 2003 (68 FR 40709).

The Commission's related evaluation of this amendment is contained in a Safety Evaluation dated November 24, 2003.

No significant hazards consideration comments received: No.

Entergy Nuclear Operations, Docket No. 50-247, Indian Point Nuclear Generating Unit No. 2, Westchester County, New York

Date of application for amendment: March 27, 2002, as supplemented on May 30, July 10, October 10, October 28, November 26, and December 18, 2002, and on January 6, January 27, February 26, April 8, May 19, June 23, June 26, July 15, August 6, September 11, October 8, and October 14, 2003.

Brief description of amendment: This amendment converts the current Technical Specifications (TS) to a set of Improved TS based on NUREG-1431, Revision 2, "Standard Technical Specifications for Westinghouse Plants," Revision 2, dated June 2001.

Date of issuance: November 21, 2003.

Effective date: As of the date of issuance to be implemented within 60 days.

Amendment No.: 238.

Facility Operating License No. DPR-26: Amendment replaced the current Technical Specifications (TSs) with the Improved TSs in their entirety and revised the license.

Date of initial notice in Federal Register: September 26, 2003 (68 FR 55660).

The supplemental letters that were received subsequent to the issuance of the **Federal Register** notice provided clarifying information that did not change the no significant hazards consideration.

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated November 21, 2003.

No significant hazards consideration comments received: No.

Entergy Nuclear Operations, Inc., Docket No. 50-293, Pilgrim Nuclear Power Station, Plymouth County, Massachusetts

Date of application for amendment: March 19, 2003.

Brief description of amendment: This amendment deletes Technical Specification (TS) 5.5.3, "Post Accident Sampling," and thereby eliminates the requirements to have and maintain the post accident sampling system at the Pilgrim Nuclear Power Station.