

standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment requests involve no significant hazards consideration.

Attorney for licensee: Christopher J. Warner, Esq., Pacific Gas and Electric Company, P.O. Box 7442, San Francisco, California 94120.

NRC Section Chief: Stephen Dembek.

Southern California Edison Company, et al., Docket Nos. 50-361 and 50-362, San Onofre Nuclear Generating Station, Units 2 and 3, San Diego County, California

Date of amendments request:

December 9, 2002.

Description of amendments request:

The proposed amendments would revise Technical Specification 3.7.5, "Auxiliary Feedwater System," Surveillance Requirement (SR) 3.7.5.2 for San Onofre Nuclear Generating Station, Units 2 and 3. Specifically, the proposed change would change wording of the Frequency of SR 3.7.5.2 from "31 days on a Staggered Test Basis" to "In accordance with the Inservice Testing Program." Such inservice tests confirm component operability, trend performance, and detect incipient failures by indicating abnormal performance. This change is requested to implement recommendations from the Standard Technical Specifications for Combustion Engineering Plants, NUREG-1432, Revision 2.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. Does the proposed change involve a significant increase in the probability or consequences of an accident previously evaluated?

Response: No.

In June 2001, the Nuclear Regulatory Commission (NRC) issued NUREG 1432, Revision 2, "Standard Technical Specifications Combustion Engineering Plants." For Technical Specification 3.7.5, "Auxiliary Feedwater (AFW) System," Surveillance Requirement (SR) 3.7.5.2 requires verification that each AFW pump's developed head at the flow test point is greater than or equal to the required developed head which ensures that AFW pump performance has not degraded during the cycle. This test confirms one point on the pump design curve and is indicative of overall performance. This proposed change will revise San Onofre Nuclear Generating Station (SONGS) Surveillance Frequency to be consistent with NUREG 1432, Revision 2. This change in and of itself will have no effect on the probability or consequences of an accident previously evaluated.

Once this change to the Technical Specification is approved, changes to the Surveillance Frequency of the AFW pumps would be controlled in accordance with the Risk-Informed Inservice Testing Program.

Therefore, the proposed change does not involve a significant increase in the probability of consequences of any accident previously evaluated.

2. Does the proposed change create the possibility of a new or different kind of accident from any accident previously evaluated?

Response: No.

The proposed amendment will not change the design, configuration or method of operation of the plant. Therefore, the proposed change does not create the possibility of a new or different kind of accident from any previously evaluated.

3. Does the proposed change involve a significant reduction in a margin of safety?

Response: No.

The proposed amendment will change the SR 3.7.5.2 Frequency from "31 days on a Staggered Test Basis" to "In accordance with the Inservice Testing Program." The proposed change does not change the operation or surveillance requirements. It does not change the design function of any of AFW system components. Therefore, the proposed change does not involve a significant reduction in the margin of safety.

Based on the above, Southern California Edison concludes that the proposed amendment present no significant hazards consideration under the standards set forth in 10 CFR 50.92(c), and, accordingly a finding of "no significant hazards consideration" is justified.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment requests involve no significant hazards consideration.

Attorney for licensee: Douglas K. Porter, Esquire, Southern California Edison Company, 2244 Walnut Grove Avenue, Rosemead, California 91770.

NRC Section Chief: Stephen Dembek.

Southern Nuclear Operating Company, Inc., Georgia Power Company, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, City of Dalton, Georgia, Docket Nos. 50-321 and 50-366, Edwin I. Hatch Nuclear Plant, Units 1 and 2, Appling County, Georgia

Date of amendment request:

December 2, 2002.

Description of amendment request:

The proposed amendments change Technical Specification Surveillance Requirement 3.6.4.1.2 to require that only one access door in each access opening of the secondary containment be verified closed every 31 days.

Basis for proposed no significant hazards consideration determination:

As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. [Does] the proposed change [* * *] involve a significant increase in the probability or consequences of an accident previously evaluated[?]

The proposed change to Surveillance Requirement SR 3.6.4.1.2 would require that only one of the two secondary containment access doors be verified closed; presently, both doors are required to be verified closed. This change is administrative in nature in that it does not involve, require, or result from any physical change to the secondary containment boundary or access door configuration. The change to Surveillance Requirement SR 3.6.4.1.2 is consistent with TSTF Standard Technical Specification Change Traveler TSTF-18, Revision 1, and Surveillance Requirement SR 3.6.4.1.3 of Revision 2 of Volume 1 of NUREG-1433. As indicated in the "Justification" portion of Standard Technical Specification Change Traveler TSTF-18, Revision 1, verifying one of the two access doors is closed is sufficient to ensure that the infiltration of outside air does not prevent the establishment and preservation of the required negative pressure within the secondary containment. Indeed, neither the requirements regarding minimum negative pressure and maximum infiltration and drawdown time nor the actions required to be taken should these requirements not be met will be altered by the proposed Licensing amendment.

Because the physical characteristics and performance requirements of the secondary containment will not be altered and the change to Surveillance Requirement SR 3.6.4.1.2 is consistent with the current revision of NUREG-1433, the proposed Licensing amendment can not involve a significant increase in the probability or consequences of any accident previously evaluated.

2. [Does] the proposed change [* * *] create the possibility of a new or different kind of accident from any previously evaluated[?]

For the reasons previously discussed, neither the secondary containment boundary nor the access door configuration will be altered by or because of the proposed change to the surveillance requirement. Likewise, the requirements defining and governing secondary containment operability and functionality, that is, Standby Gas Treatment system flow rate and secondary containment negative pressure and drawdown limits, will not be changed. The secondary containment, including its access openings, will remain physically unaltered; will function as presently described in the Updated Final Safety Analysis Report [(UFSAR)]; and will be subject to the same structural and functional requirements. Under these circumstances, this change can not, and does not, create the possibility of a new or different kind of accident from any previously evaluated.

3. [Does] the proposed change [* * *] involve a significant decrease in the margin of safety[?]